# U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT

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$ \begin{array}{c} \begin{array}{c} \text{CONTROL BLOCK} \ \underline{///////} (1) \ (\text{PLEASE PRINT OR TYPE ALL RTQUIRED INFORMATION}) \\ \underline{/0/1/} \ \underline{/V/A/N/A/S/2/} (2) \ \underline{/0/0/-/0/0/0/0/-/0/0/} (3) \ \underline{/+1/1/1/1} (4) \ \underline{///} (5) \\ \underline{\text{LICENSEE CODE}} \ \underline{\text{LICENSE NUMBER}} \end{array} $			
$\frac{D/1/2}{SOURCE} \xrightarrow{/L/2} (6) \xrightarrow{/0/5/0/0/3/3/9/2} (7) \xrightarrow{/0/6/2/6/8/0/2} (8) \xrightarrow{/0/2/2} (9) \xrightarrow{/0/2} (9)$			
EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)			
/0/2/ / On June 26, 1980, with the unit in Mode 3, while performing channel calibra- /			
<pre>/0/3/ / tions on the pressurizer press. protection channels, it was found that the /</pre>			
/0/4/ / pressurizer protection Ch. I, II and III indicated values below actual RCS /			
10/5/ / pressure. This is contrary to T.S. 2.2.1 and T.S. 3.3.2.1. Since the fission /			
<pre>/0/6/ / product inventory is negligible and the pressurizer press. control channels /</pre>			
<pre>/0/7/ / were indicating the correct RCS pressure, the health and safety of the general /</pre>			
/0/8/ / public were not affected. Reportable pursuant to T.S. 6.9.1.8.b. /   SYSTEM CAUSE COMP. VALVE   CODE CODE SUBCODE COMPONENT CODE SUBCODE			
/0/9/ /I/A/ (11) /E/ (12) /B/ (13) /I/N/S/T/R/U/ (14) /T/ (15) /Z/ (16)			
LER/RO EVENT YEAR REPORT NO. CODE TYPE NO. (17) REPORT			
NUMBER /8/0/ /-/ /0/3/1/ /// /0/1/ /T/ /-/ /0/			
ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB. SUPPLIER MANUFACTURER			
<u>/E/ (18) /E/ (19) /Z/ (20) /Z/ (21) /0/0/0/ (22) /Y/ (23) /N/ (24) /N/ (25) /1/2/0/4/ (26</u>			
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)			
/1/0/ / The cause of the low pressure shift in the transmitters is due to a "pressure /			
/1/1/ / set" of the bourdon tube within the transmitters which occurs after the first /			
/1/2/ / prolonged operation at normal operating pressure. The transmitters were cali- /			
/1/3/ / brated upon discovery of this event and will be calibrated periodically to /			
/1/4/ / determine any additional shift in output. /			
FACILITY METHOD OF STATUS %POWER OTHER STATUS DISCOVERY DESCRIPTION (32)			
/1/5/ /X/ (28) /0/0/0/ (29) / HOT STANDBY/ (30) /B/ (31) / OPERATOR OBSERVATION /			
ACTIVITY CONTENT			
RELEASEDOF RELEASEAMOUNT OF ACTIVITY (35)LOCATION OF RELEASE (36)/1/6//2/ (33)/2/ (34)/ NA/NA			
PERSONNEL EXPOSURES			
NUMBER TYPE DESCRIPTION (39)   /1/7/ /0/0/0/ (37) /Z/ (38) / NA /   PERSONNEL INJURIES / / /			
NUMBER DESCRIPTION (41)			
/1/8/ /0/0/ (40) / NA LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION (43)			
<u>/1/9/ /Z/ (42) / NA</u> PUBLICITY //			
ISSUED DESCRIPTION (45) NRC USE ONLY   /2/0/ /N/(44) /NA ////////////////////////////////////			
80071508/7 NAME OF PREPARER W. R. CARTWRIGHT PHONE (703) 894-5151			

Virginia Electric and Power Company North Anna Power Station, Unit <sup>12</sup> Docket No. 50-339 Report No. LER 80-031/01T-0

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### Description of Event

On June 26, 1980, with the unit in mode 3, while performing a periodic test on the auxiliary shutdown panel, it was discovered that the pressurizer pressure protection channels disagreed with the pressurizer pressure control channels. Protection Channel I was immediately placed "in test" and transmitter PT-2455 was calibrated. It was found that this transmitter was reading 35.4 psig below actual KCS pressure. Subsequently, Channel II and Channel III were calibrated and they indicated 40.0 psig and 41.6 psig low, respectively.

This is contrary to T.S. 2.2.1 (Table 2.2-1) since the maximum allowable high pressure reactor trip setpoint of 2395 PSIG was exceeded and T.S. 3.3.2.1 (Table 3.3-3) since the maximum allowable setpoint of 2010 psig for the P-11 interlock was exceeded.

The following is a table showing actual as found setpoints of the three pressurizer pressure protection channels for pressurizer pressurehigh and P-11 setpoints.

PZR Pressure Channel (Protection)	Reactor Trip (PSIG)	P-11 Psig
I	2420.4	2015.4
II	2425.0	2020.0
III	2426.6	2021.6

Initially this event was classified as a 30 day report; upon further evaluation on July 1, 1980, it was determined to be 14 day reportable pursuant to T.S. 6.9.1.8.b.

### Probable Consequences of Occurrence

The consequences of having all three pressure protection channels drift low by an average of 39 psi was to effectively decrease the margins used in the safety analysis by requiring higher RCS pressures to initiate a reactor trip and allowing manualy blocking of the low pressure initiated safety injection signal at higher pressures. Since the pressurizer pressure control channels were reading correctly and the event was discovered prior to any significant power levels of the reactor plant, the health and safety of general public were not affected.

# Cause of Event

The cause of this event is thought to be a "pressure set" condition of the bourdon tube internal to the pressure transmitter. The fact that after being subjected to 2235 psig for approximately two months, all these transmitters of this design indicated an average of 39 psig below RCS pressure with a standard deviation of only 3.2 psi. The transmitters used in the control channels are of a different design and manufacturer and did not exhibit this trait.

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Westinghouse Nuclear Service Division by way of memorandum has confirmed that this problem has existed on other plants using the Barton Pressurizer Pressure Transmitters. After approximately two months of operation these transmitters were found to have a 2.5 to 4.0% low shift in their pressure output signal. After recalibration, however, the "pressure set" stabilizes and shows no deviation upon additional recalibration checks.

#### Immediate Corrective Action

Upon discovery of a deviation between reading from the pressure protection channels and pressure control channels, a deviation r. t was written and Protection Channel I was placed in test and recalibrated. At this time it was unknown whether the protection or control channels were indicating incorrected. Upon further investigation, it was discovered that the protection channels were providing the incorrect values. As soon as Channel I was calibrated it was placed in service and Channel II was placed "in test" and calibrated. Channel III was likewise recalibrated.

### Schduled Corrective Action

Periodic checks will be made on the output of the pressure transmitter to ensure that they have stablized.

# Actions Taken to Prevent Recurrence

The performance of periodic testing to check the calibration of the transmitters will prevent recurrence of this event. Control room logs taken every eight hours will be reviewed to compare pressure control channels with pressure protection channels to determine any variance in readings.

# Generic Implications

There appears to be a generic problem with the Barton transmitters used at North Anna. This is documented by Westirghouse in correspondence to Vepco. However, the problem appears to exist only when the transmitters are initially put into service. Based on experience with unit 1, the transmitters perform properly after the initial "pressure set".