

U.S. NUCLEAR REGULATORY COMMISSION
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-155/80-06

Docket No. 50-155

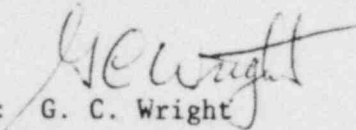
License No. DPR-6

Licensee: Consumers Power Company
212 West Michigan Avenue
Jackson, MI 49201


Facility Name: Big Rock Point Nuclear Plant

Inspection At: Charlevoix, MI

Inspection Conducted: May 5-8, 1980

Inspector:  G. C. Wright

5/23/80

Approved By:  K. R. Baker, Acting Chief,
Projects Section 3-2

5/23/80

Inspection Summary

Inspection on May 5-8, 1980 (Report No. 50-155/80-06)

Areas Inspected: Routine, unannounced inspection pertaining to followup of items in NUREG 0578 identified during meeting on February 28 & 29, 1980 between DOR and the licensee (refer to Inspection Report 80-03). The inspection involved 17 hours onsite by one NRC inspector.

Results: Of the areas inspected no items of noncompliance were identified.

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DETAILS

1. Persons Contacted

- *A. Sevener, Operations Supervisor
- *R. Schroder, Technical Superintendent
- *C. Axtell, Plant Health Physicist
- *D. DeMoor, Technical Engineer
- *T. Fisher, QA Analyst
- C. Hartman, Plant Superintendent

The inspector contacted several other licensee employees, including shift supervisors and operations personnel.

*Denotes those attending the exit interview on May 8, 1980.

2. Review of Items Pertaining to NUREG 0578

The sections referenced in the following paragraphs refer to sections in NUREG 0578.

- a. Section 2.1.1: The inspector reviewed the following procedures and verified that methods exist to transfer air supply to RDS valves from their normal power supply to the diesel generator.
 - 1. ONP 2.36, "Loss of Station Power"
 - 2. SOP 28 Rev 7, 4/16/79 "Station Power Supply."

- b. Section 2.1.3.a:
 - 1. The inspector reviewed procedure ALP16, Rev. 0, 1/11/80 "Steam Drum Relief Valve Monitor Annunciator" and verified that adequate information is given to the operator regarding positive valve position indication system for the relief valves.
 - 2. The inspector reviewed procedures ALP16, "Drum Safety Valve Leak" and ALP4, window 34 "Drum Safety Valve Leak" to insure that a backup means of relief valve position verification is adequately addressed.

- c. Section 2.1.3.b:
1. The inspector reviewed procedure SOP8, Rev. 12, 4/15/80, "Post Incident System: and verified that a means exists to switch the core spray flow indicator input from one transmitter to the other transmitter.
 2. The inspector reviewed procedure SOP18, Rev. 10, 4/10/80 "Reactor Depressurization System" and verified that before manual initiation of the RDS system the operator verify that the core spray header pressure is available.
- d. Section 2.1.4:
- The inspector reviewed the following procedures and verified that adequate instruction and precaution have been given to the operators about the containment isolation logic modification.
1. GOP4, Rev. 7, 2/26/80 "Reactor Trip Recovery"
 2. ONP2.31, Rev. 5, 3/20/80 "Reactor Scrams"
- e. Section 2.1.6.a:
- The inspector reviewed the following procedure relative to the core spray leakage reduction program.
1. TR-05, Rev. 7, 12/21/79 "Core Spray Pump Run and Test Loop Operation"
 2. T-3-14, Rev. 6, 3/27/80 "Monthly Core Spray Heat Exchanger Leak Test"
- f. Section 2.1.8.a:
1. The inspector reviewed procedure EP2, Rev. 1, 1/12/80 "Sampling of Core Spray Heat Exchanger" for technical and precautionary content.
 2. The inspector reviewed procedure EP C-4, "Procedure to Determine the Extend of Core Damage" of volume 9 "Site Emergency Plan" Appendix C and verified that adequate information is given personnel in the use of containment air volume radiation monitor.

- g. Section 2.1.8.b:
1. The inspector reviewed procedure EP C-2 of Volume 9 "Procedure to Determine High Stack Gas Releases" of volume 9 "Site Emergency Plan" Appendix C and verified that adequate information is available for use of the high range noble gas stack monitor.
 2. The inspector reviewed procedure EP1, Rev. 1, 1/10/80 "Airborn Iodine Monitoring under Accident Conditions in-plant" and verified adequate instruction is provided for the use of the air samplers, silver-zeolite cartridges, and count rate devices.
- h. Section 2.1.8.c:
- The inspector verified that an emergency kit is in the Technical Support Center and that it includes air sample pump, silver-zeolite cartridges and filters.
- The inspector determined that the licensee does not maintain an emergency kit in the operations support center but does have the air samplers, silver-zeolite cartridges and filters available at Access Control.
- i. Section 2.2.1.a:
- The inspector confirmed the licensee's commitment to revise Procedure 1.4 in the Administrative Procedure volume to require the shift supervisors presence in the control room at all times during a "site" or general emergency. (Reference Consumers Power letter to Mr. Zieman of March 14, 1980).
- j. Section 2.2.1.b:
- The inspector reviewed the following procedures relative to implementation and organization of the Shift Technical Advisors Program.
1. Administrative Procedure Chapter 2 Section C.4.2.2.
 2. Site Emergency Plan volume 9 Individual Instruction Card 7-A.
 3. Master Training Manual volume 18, Chapter 2, Section 5.6.

- k. Section 2.2.2.a: The inspector reviewed the following procedures governing control room access and verified adequate control of personnel access is indicated.
 - 1. Administrative Procedures Chapter 4, Section A.1.7 "Control Room Access"
 - 2. Site Emergency Plan volume 9 section "Site Emergency Director Instructions"

- i. Section 2.2.2.b&c: The inspector reviewed the following procedures pertaining to the use, initiation and manning requirements for both the Technical and Operations Support Centers.
 - 1. Site Emergency Plan volume 9, Section 9.4 "Implementation of Plan"
 - 2. Site Emergency Plan volume 9, section 9.8 "Emergency Equipment and Facilities"

No items of noncompliance were identified.

3. Open Items for I&E Review from NUREG 0578

- a. Review installation package for steam drum safety valve position indication instrumentation.
- b. Environmental qualification of steam drum safety valve position indication instrumentation.
- c. Low noise level alarm modification on the steam drum safety valve position indication instrumentation.
- d. Review installation of containment isolation logic modification.
- e. Review installation of containment air volume radiation monitor.
- f. Review installation of high range noble gas stack radiation monitors.

The above items will be inspected during a subsequent inspection (155/80-06-1,a-f).

4. Exit Interview

The inspector met with the licensee representatives (denoted in paragraph 1) at the conclusion of the inspection on May 8, 1980. The inspector summarized the scope and finding of the inspection.