INTERIM REPORT

8005130445 LPDR

Accession No.

Contract Program or Project Title: Light Water Reactor Thermal Hydraulic Development Program

Subject of this Document:

February Monthly Fighlight Letter

Type of Document:

Monthly Highlight Letter

Author(s):

Owen C. Jones, Jr. Department of Nuclear Energy Brookhaven National Laboratory Upton, New York 11973

Date of Document:

Responsible NRC Individual and NRC Office or Division: February 1980

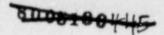
Dr. Y. Y. Hsu Division of Reactor Safety Research Systems Engineering Branch U.S. Nuclear Regulatory Commission Washington, D.C. 20555

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

> Brookhaven National Laboratory Upton, NY 11973 Associated Universities, Inc. for the U.S. Department of Energy

Prepared for U.S. Nuclear Regulatory Commission Washington, D. C. 20555 Under Interagency Agreement DE AC02-76CH 00016 NRC FIN No. A- 3045

> INTERIM REPORT NRC Research and Technical Assistance Report



Monthly Highlights

for

February 1980\*

Light Water Reactor Thermal Hydraulic Development Division

Owen C. Jones, Jr. Thermal Hydraulic Dr relopment Division Department of Nuclear Energy Brookhaven National Laboratory Upton, New York 11973

\* Work carried out under the auspices of the United States Nuclear Regulatory Commission.

> NRC Research and Technical Assistance Report

## 2. Nonequilibrium Phase Change Studies

## 2.1 Development of Analytical Modeling (N. Abuaf, B.J.C. Wu)

The flashing onset correlation developed by Alamgir and Lienhard for static depressurization was compared with the depressurization data available from the Marviken critical flow tests. The results for the 26 tests examined are presented in Fig. 1. The pressure difference between the initial pressure for each test and the turning point in the pressure history,  $p_0 - p_N$ , are plotted vs. the depressurization rate. The depressurization rates recorded in the Marvikon tests are below the limit of applicability of the Alamgir-Lienhard correlation and this is the region where the data avrilable in the literature showed a wide unexplainable scatter. Although the extrapolated Alamgir-Lienhard correlation predicts a constant pressure undershoot for  $T_0 = 264$  C, for the depresurization range considered, the Marviken data depicts an increase in the undershoot with the depressurization rate which is the normally expected trend. The dissolved oxygen content of the Marviken data for the measured cases ranges from 0.005 -1.3 mg 02/kg H20 and the data does not show a trend with the dissolved 02 content. The 1.3 mg 02/kg H20 would correspond to approximately 1 cc of 02 at standard conditions per kg of water, which could be very low in order to cause a considerable effect on the inception point. The typical reactor water in PWR and BWR's contain 0.25 - 0.43 ppm, of 02, and thus the dissolved oxygen in reactor water would probably not to change the inception. The H<sub>2</sub> concentration on the other hand were quoted to be 15 - 35 cc/kg of H<sub>2</sub>O in PWR's and such quantities of dissolved H2 may have an effect on the inception points.

The variation of the observed pressure undershoot with the depressurization rates and the absence of any observed effect of the 02 concentrations seem to reinforce the necessity of conducting shock tube experiments as proposed earlier

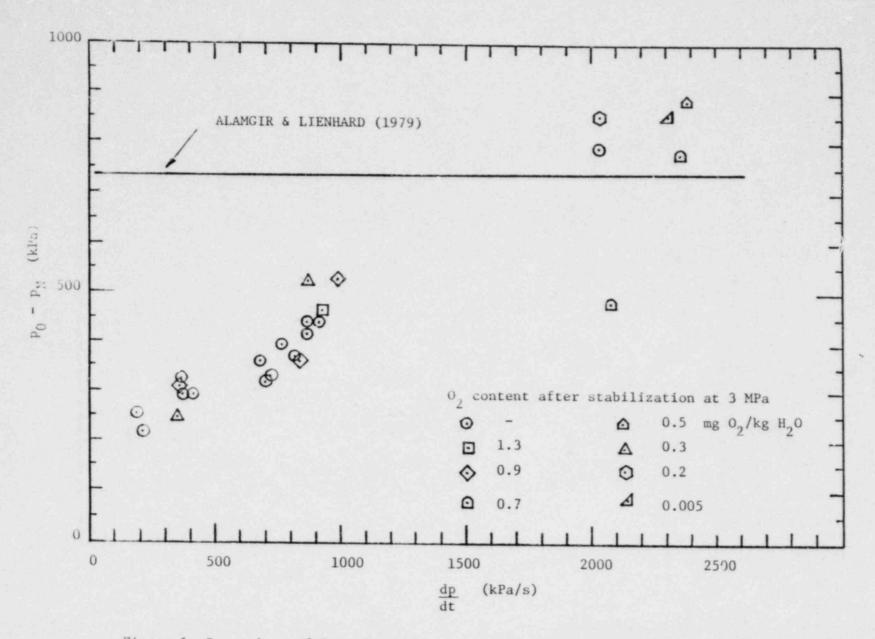


Figure 1 Comparison of Experimental  $p_0 - p_N$  with Alamgir-Lienhard Correlation. (Marviken Tests 1-27)

to determine the onset points at low depressurization rates and systemmatically study the effect of various dissolved gases and particulates.

The methodology developed to calculate the critical mass flow rates through converging nozzles from nozzle geometry and inlet conditions  $(p_0, T_0)$  was successfuly applied in predicting Powell's data.

## 2.2 Flashing Experiments (G.A. Zimmer, J.H. Klein, B.J.C. Wu, and N. Abuaf)

The multichannel scanner which is the basis of the automatic data acquisition system broke down during the calibration of the thermocouples, pressure transducers and the five beam gamma densitometer. The scanner was replaced. The wiring of the new scanner is being accomplished in order to finish the calibration and start the flashing experiments.

The void fraction data obtained in the last series of runs is being analyzed and the necessary corrections and uncertainty calculations are being incorporated in the data analysis programs. Distribution Light Water Reactor Thermal Hydraulic Development Program

BNL RSP Division Heads BNL RSP Group Leaders BNL RSE Personnel

.

S. Fabic, NRC
Y. Y. Hsu, NRC
W. Y. Kato, BNL
H. J. Kouts, BNL
R. M. Scroggins, NRC
T. G. Theofanus, Purdue University
N. Zuber, NRC

U.S. NRC Division of Technical Information and Control