

LICENSEE EVENT REPORT

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME										LICENSE NUMBER										LICENSE TYPE					EVENT TYPE				
1	A	R	A	R	K	I				0	0	-	0	0	0	0	0	-	0	0	4	1	1	1	1		0	3	
8	9						14			15										25	26						30	31	32

CATEGORY		REPORT TYPE	REPORT SOURCE	DOCKET NUMBER							EVENT DATE					REPORT DATE								
0	1	CON'T	0	L	0	5	0	-	0	3	1	3	0	1	2	8	7	7	0	4	0	8	7	7
7	8	57	58	59	60	61						68	69					74	75					80

EVENT	DESCRIPTION
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100	100.0000

02 AT HOT SHUTDOWN CONDITIONS, DURING A ROUTINE WALK-THROUGH INVESTIGATION, 80
7 8 9

03 A PINHOLE LEAK WAS OBSERVED ON THE PIPE-TO-FLANGE WELD BETWEEN VALVE 80
7 8 9

04 RBY-77B AND "B" RCP SEAL. THIS EVENT WAS A REPETITIVE OCCURRENCE, 80
7 8 9

05 (SO-313/77-5A) 80
7 8 9

06 80
7 8 9

7 8 9 10 11 12 13 14 15 16 17 43 44 45 46 47 48

SYSTEM CODE CAUSE CODE COMPONENT CODE PRIME COMPONENT SUPPLIER COMPONENT MANUFACTURER VIOLATION

07 CB F PIPEXX Z Z a a u N

CAUSE DESCRIPTION

08	7	8	9	NA	80
09	7	8	9	THE DEFECTIVE WELD WAS REPAIRED DURING THE RECENT REFUELING	80
10	7	8	9	OUTAGE.	80

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
11	H	000		NA		b		NA	
7	8	9	10	11	12	13	44	45	46
FORM OF ACTIVITY RELEASED		CONTENT OF RELEASE		AMOUNT OF ACTIVITY				LOCATION OF RELEASE	
12	E	E		NA				NA	
7	8	9	10	11	12	13	44	45	46

PERSONNEL EXPOSURES

NUMBER				TYPE	DESCRIPTION
1	3	0	0	0	NA

PERSONNEL INJURIES

NUMBER				DESCRIPTION
1	4	0	0	0 NA

OFFSITE CONSEQUENCES

15 7 8 9 20

LOSS OR DAMAGE TO FACILITY

TYPE		DESCRIPTION
16	3	NA

PUBLICITY

17

ADDITIONAL FACTORS

1A

19 | 8004140743

NAME: DAVID G. MARDIS PHONE: 501/371-4496

1. Reportable Occurrence Report No. 50-313/77-5A
2. Report Date: 4-8-77 3. Occurrence Date: 1-28-77

4. Facility: Arkansas Nuclear One-Unit 1
Russellville, Arkansas

5. Identification of Occurrence:

A leak on the pipe to flange weld between valve RBV-77B and "B" RCP seal.

6. Conditions Prior to Occurrence:

Steady-State Power _____	Reactor Power _____ 0 _____ MWth
Hot Standby _____	Net Output _____ 0 _____ MWe
Cold Shutdown _____	Percent of Full Power _____ 0 _____ %
Refueling Shutdown _____	Load Changes During Routine Power Operation _____
Routine Startup Operation _____	
Routine Shutdown Operation _____	
Other (specify) Hot Shutdown	

7. Description of Occurrence:

At approximately 0200 hours on January 28, 1977, during a routine walk-through inspection by Operations Personnel prior to cooldown, a pinhole leak was observed on the pipe-to-flange weld between Valve RBV-77B and "B" RCP seal. Valve RBV-77B serves as a means of venting the "B" RCP seal during initial fill of the RCS. The leak could not be isolated.

8. Designation of Apparent Cause of Occurrence:

Design	_____	Procedure	_____
Manufacture	_____	Unusual Service Condition Including Environmental	_____
Installation/ Construction	_____	Component Failure (See Failure Data)	_____
Operator	_____		
Other (specify)	X		
Unknown			

9. Analysis of Occurrence:

The Reactor Coolant System leakrate prior to shutdown was approximately 0.5 GPM. Since the leakage was contained in a controlled area, there was no hazard to the health and safety of the public.

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10. Corrective Action:

The defective weld was repaired during the recent refueling outage by grinding and rewelding.

11. Failure Data:

This is the second failure in this area. See Abnormal Occurrence Report 74-14.