SMUD

September 26, 1975

Mr. R. H. Engelken
Director of Regulatory Operations, Region V
NRC Operations Office
1990 N. California Boulevard
Walnut Creek Plaza, Suite 202
Walnut Creek, CA 94596

Re: Operating License DPR-54
Docket No. 50-312
Abnormal Occurrence 75-12



Dear Mr. Engelken:

As defined by Proposed Amendment No. 30 to Technical Specifications for the Rancho Seco Nuclear Generating Station, Section 6.9.1, and Regulatory Guide 1.16, Revision 3, Section C.2.B.2, we are reporting the following Abnormal Occurrence as AO-75-12.

Excessive air leakage of the Reactor Building Purge Outlet Penetration was detected August 27, 1975, while the Surveillance Procedure for local leak rate testing was being performed. The plant was in a cold shutdown condition for maintenance during this period. Initial attempts to pressurize the penetration for measuring the leak rate failed, and leakage was detected through the 66 inch inner butterfly valve.

The valve seats on the inner valve were adjusted, and subsequent testing revealed leakage from the penetration of greater than 300,000 standard cc per minute, with a penetration pressure of 10 psig. Total leakage allowable from all penetrations inspected during a local leak rate test, as stated in Technical Specifications Section 4.4.1.2.3, is .06 percent of the Reactor Building atmosphere every 24 hours. This limit converted to a leakage rate is 106,600 standard cc per minute at the minimum allowable test pressure (52 psig).

Further adjustments were made to the inner valve seats which reduced leakage through that valve essentially to zero, as verified by local bubble testing. The penetration leakage was measured at 33,550 standard cc per minute, which was through the outer penetration valve. Attempts to reduce leakage by adjustment of the valve seats were not successful, so the seats will be replaced and adjusted to give a proper seal. To assure that the leakage through the penetration remains within acceptable limits, the surveil-lance procedure will be repeated prior to reactor startup which is scheduled for November 7, 1975. A thin film of silicon grease will periodically be applied to the seats to improve the seal.

September 26, 1975 Mr. R. H. Engelken -2-The purge valves are 66 inch butterfly valves, Model 60 WR manufactured by Allis Chalmers Company. There was no plant transient associated with this event. Respectfully, Assistant General Manager and Chief Engineer Enclosure ccs: Director, MIPC(3) Director, IE(30)

## LICENSEE EVENT REPUHI

LICENSEE EVENT REPURT
CONTROL BLOCK:
UCENSEE NAME  C  A R  S  S  1    0  0  -  0  0  0  0  -  0  0    4  1  1  1  1  1  1  1  1  1  1  1  1  1
14   15     15
EVENT DESCRIPTION  While performing the local leak rate test on the Reactor Building Purge Outlet Valve  80
While performing the local leak rate test on  Penetration, excessive leakage was found through the inner valve. The outer valve  Penetration, excessive leakage was found through the inner valve seats were adjusted to  was found to have an acceptable leak rate, the inner valve seats were adjusted to  80
was found to have an acceptance of the second secon
SYSTEM CAUSE COMPONENT CODE COMPONENT SUPPLER A 1 8 0 N 48
CAUSE DESCRIPTION  CAUSE DESCRIPTION  66 inch valve improperly adjusted. Seats were adjusted to give 80
Valve seats on 05 Inch  B 9  proper seal. Surveillance will be performed in six months to verify seaf Inch  BO  METHOD OF DISCOVERY DESCRIPTION
STATUS  STATUS  G  G  10  10  12  13  OTHER STATUS  DISCOVERY  N. A.  B  N. A.  B  LOCATION OF RELEASE
FORM OF ACTIVITY OF RELEASE N.A. AMOUNT OF ACTIVITY N.A. 80
NUMBER TYPE DESCRIPTION N.A.
PERSONNEL STUTION NUMBER T SCRIPTION BO
14 0 0 0 0 N. A.  OFFSITE CONSEQUENCES  60
15 N. A.  1 8 9  LOSS OR DAMAGE TO FACILITY
TYPE DESCRIPTION N.A.
PUBLICITY    Sacramento Union and Sacramento Bee notified 9/4/75.   Sacramento Union and Sacramento Bee notified 9/4/75.   17 8 9
ADDITIONAL FACTORS  80
7 8 9
19   PHONE: (916) 452-3211   SPO 881-66