



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report Nos. 50-348/80-10 and 50-364/80-11

Licensee: Alabama Power Company
600 North 18th Street
Birmingham, AL 35202

Facility Name: Farley

License Nos. NPF-2 and CPPR-86

Inspection at Farley site near Dothan, Alabama

Inspector: *W. H. Bradford for*
W. H. Bradford

May 21, 1980
Date Signed

Approved by: *R. D. Martin for*
R. D. Martin, Section Chief, RONS Branch

May 21, 1980
Date Signed

SUMMARY

Inspection on March 31 through April 25, 1980

Areas Inspected

This routine, announced inspection involved 155 inspector-hours on site in the areas of hot functional testing, pre-operational testing, unit no. 2 plant testing status, unit no. 2 plant tour, unit no. 1 plant operation, review of licensee event reports, IEB followup, plant security, open items, and review of administrative controls.

Results

Of the 10 areas inspected, no items of noncompliance or deviations were identified.



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DETAILS

1. Persons Contacted

Licensee Employees

W. G. Hairston, Plant Manager
J. D. Woodard, Assistant Plant Manager
W. D. Shipman, Maintenance Superintendent
D. Morey, Operations Superintendent
R. S. Hill, Operations Supervisor
R. W. McCracken, Technical Superintendent
D. E. Mansfield, Startup Superintendent, Unit No. 2
R. M. Coleman, Supervising Engineer
H. M. McClelland, Plant Engineer
K. W. Kale, QA Engineer
R. E. Hollands, QA Supervisor (Unit No. 2)

Other licensee employees contacted included shift supervisors, shift foremen, plant operators, security force members, plant technicians and office personnel.

Other Organizations

S. M. Hall, Westinghouse Startup Manager and certain Westinghouse startup engineers.

2. Exit Interview

The inspection scope and findings were summarized during management interviews on April 11 and 25, 1980, with the plant manager and selected members of his staff. The licensee acknowledged the inspection findings.

3. Licensee Action on Previous Inspection Findings

Not inspected.

4. Unresolved Items

Unresolved items were not identified during this inspection.

5. Witness of Integrated Hot Functional Testing

The inspector witnessed the "Hot Functional Testing" as controlled by test procedure 064-6-016 "Hot Functional Outline". Testing was witnessed when the primary coolant system was at ambient temperature to the 547° test plateau when the primary and secondary systems were at rated temperature and pressure conditions.

The inspector verified that the appropriate approved procedures were in use, listed prerequisites were being met, the required plant support systems were in service, test crew actions were correct and timely, and the required data was being collected and recorded as required by the various test procedures.

The inspector verified that test personnel were adhering to the licensee's administrative practices and were maintaining proper test discipline concerning test execution, test procedure changes and test records.

Control room shift logs were reviewed to verify that a complete and accurate record of the testing in progress was being maintained. The inspector also observed the control room activities to verify that proper control of the various systems were being maintained.

Within the areas inspected the inspector had no adverse comments or questions regarding the conduct of the "Hot Functional Testing."

6. Witness of Preoperational Testing

Portions of the following preoperational tests were witnessed by the inspector. The test results were reviewed to verify that acceptance criteria was met. The performance of each test was evaluated against the requirement of ANSI N18.7-1972, section 6.0, "Test and Inspection Procedures", ANSI N45.2-1971, section 12, "Test Control", and FSAR Chapter 14, "Initial Tests and operation

- 100-5-003 "Startup Transformer Full Load Preoperational Test"
- 040-5-017 "RCP Seal Injection and Charging Pump"
- 040-5-021 "Solid System Pressure Control"
- 064-5-015 "RTD/1. Cross Calibration"
- 040-5-018 "BTRS Preop"
- 083-5-003 "Steam Dump Preop"
- 093-3001 "Turbine Trip Verification Test"
- 100-5-004 "Hot Functional Piping Thermal Expansion Test"
- 043-4-003 "Condenser Air Removal Acceptance Test"
- 040-5-015 "CVCS Letdown Preop"
- 055-3-001 "CRDM MG Set Functional Test"

- 358-3-3-01 "Reactor Trip Breakers Functional Test"
- 060-5-010 "Containment Ventilation Hot Testing"

Within the areas inspected there were no discrepancies identified.

7. Unit No. 2 Plant Testing Status

The status of the licensee's Phase II Preoperational testing program was reviewed and is summarized below:

Testing Completed	17%
Testing in Progress	37%
Testing not Started	46%

A review of turnover records of systems remaining to be turned over to startup from construction revealed that 80% of the systems or portions of systems have been turned over to startup for testing.

8. Plant Tour - Unit No.2

The inspector conducted daily tours of the unit no. 2 reactor containment building, auxiliary building and turbine building during the conduct of the Hot Function Testing program, to observe the following activities:

- a. Thermal expansion measurements at each temperature plateau were conducted.
- b. To observe the preoperational testing in progress.
- c. That combustibles near hot surfaces had been removed.
- d. That fire protection equipment was in place and had not been damaged.
- e. That housekeeping practices had been instituted and observed.
- f. That equipment undergoing testing was properly tagged.
- g. That installed instrumentation was protected and had not been abused.
- h. That deficiencies identified during testing were entered on a control list.

Within the areas inspected there were no deficiencies identified.

9. Unit No. 1 Plant Operation

The inspector reviewed plant operation to ascertain conformance with regulatory requirements, technical specification and Administrative Procedure No. 16, "Conduct of Operation - Operation Group". Station logs, such as the shift supervisor, shift foreman, control room operators, shift turnover, the out of service equipment log, night order log book, and the

limited condition of operation log were reviewed. The inspector conducted tours of selected areas of the plant which included portions of the auxiliary building, diesel generator building, and the turbine building. Observations were made of plant operations, monitoring instrumentation, radiation controls, fluid leaks, piping vibration, pipe hangers, certain valve positions, housekeeping, fire hazards, and equipment tagged out of service for maintenance. Discussions were held with plant operators throughout the plant concerning certain alarm functions and plant operations.

Within the areas inspected no items of noncompliance or deviations were identified.

10. Review of Nonroutine Events Reported by the Licensee

The following licensee events reports were reviewed for potential generic problems, to determine trends, to determine whether the information included in the report meets the NRC reporting requirements, and to consider whether the corrective action discussed in the report appears appropriate. Licensee action with respect to selected reports was reviewed to verify that the events were reviewed and evaluated by the licensee as required by the Technical Specifications, that corrective action was taken by the licensee, and that safety limits, limiting safety settings, and limiting conditions of operation were not exceeded. The inspector examined selected plant operations review committee minutes, incident reports, logs and records, and interviewed selected personnel.

LER-80-01 - Fuel Storage Pool Area Radiation Monitor (R-5) inoperable

LER-80-02 - Diesel Generator fuel oil day tank 1C level below technical specification limits.

LER-80-03 - Surveillance test not performed within required surveillance interval

LER-80-04 - Fuel storage pool area radiation monitor (R-25B) inoperable

LER-80-05 - 1B Hi-Hi steam generator level logic inoperable

LER-80-06 - Containment atmospheric radioactivity monitor (R-11) inoperable.

LER-80-07 - "B" train service water inoperable

LER-80-08 - Component cooling water hydraulic snubber CW-4-R121 inoperable

LER-80-11 - Containment isolation valve 1-SW-MOV-3134 inoperable

LER-80-15 - Malfunction of DEH system causing Tavg decrease to 539°F.

LER-80-16 - Turbine driven auxiliary feedwater pump inoperable due to high thrust bearing temperature

LER-80-17 - Turbine driven auxiliary feedwater pump inoperable due to overspeed trip.

11. Inspection and Enforcement Bulletins

a. IE Bulletin 79-21 - Temperature Effects on Level Measurements

The inspector verified by review of the licensee's response dated November 1, 1979, and through discussions with on-site management representatives that the corrective actions taken were responsive to the requirements of IE Bulletin No. 79-21.

12. Plant Security

The inspector verified that physical barriers were intact and that gates into the protected areas were closed and locked if not attended, that doors into vital areas were closed and locked or attended, and that isolation zones were free of visual obstruction.

The inspector verified that access into protected areas was controlled, that persons and packages were identified and prior to entry into protected areas, and that all persons, packages and vehicles were searched prior to entry in accordance with regulatory requirements and security procedures.

Within the areas inspected, no deviations or items of noncompliance were identified.

13. Open Items

(Closed) - Open item (79-37-02) This open item concerned revising Administrative Procedure No. 16, "Conduct of Operation - Operations Group" to include shift turnover log instructions. The inspector verified by review of AP-16 that the procedure has been revised to include instruction pertaining to the shift turnover logs and that all operating personnel on shift are required to properly complete a shift turnover log at the conclusion of each shift.

14. Review of Administrative Controls Concerning Defeat of Safety Actuation Signals During Containment Purging.

The inspector verified that the licensee had received the NRR generic letter concerning the above subject. The licensee's letters of response were also reviewed.

The licensee has performed a review of the design of the safety actuation signal circuits for the Containment Purge System and has determined that the overriding of the safety actuation signal for those particular valves is not possible. The licensee's Emergency Operating Procedure EOP-1 "Loss of Reactor Coolant" and EOP-2.0 "Loss of Secondary Coolant" requires the operator to check the containment purge isolation hand switches in the closed position.