#### **OPERATING DATA REPORT**

DOCKET NO.	50-317
DATE	6/12/80
COMPLETED BY	S.D.Merson
TELEPHONE	301-234-5240

#### **OPERATING STATUS**

1. Unit Name: <u>Calvert Cliffs No. 1</u> 2. Reporting Period: <u>May</u> , 1980	the second se
3. Licensed Thermal Power (MWt): 2,700	
4. Nameplate Rating (Gross MWe): 918	
5. Design Electrical Rating (Net MWe):845	
6. Maximum Dependable Capacity (Gross MWe):845	
7. Maximum Dependable Capacity (Net MWe):810	

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe):

10. Reasons For Restrictions, If Any:

	This Month	Yrto-Date	Cumulative
11. Hours In Reporting Period	744	3,647	44,412
12. Number Of Hours Reactor Was Critical	636.1	3,083.0	35,522.6
13. Reactor Reserve Shutdown Hours	19.3	38.3	1,098.8
14. Hours Generator On-Line	629.6	3,013.6	34,715.6
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,648,361	6,795,778	82,132,767
17. Gross Electrical Energy Generated (MWH)	526,927	2,148,392	26,960,922
18. Net Electrical Energy Generated (MWH)	502,342	2,034,231	25,683,437
19. Unit Service Factor	84.6	82.6	78.2
20. Unit Availability Factor	84.6	82.6	78.2
21. Unit Capacity Factor (Using MDC Net)	83.4	68.9	71.4
22. Unit Capacity Factor (Using DER Net)	79.9	66.0	68.4
23. Unit Forced Outage Rate	15.4	6.0	8.8
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24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

No. 1 Plant scheduled for a planned outage starting 10/17/80 until 12/15/80 for refueling, unit general inspect on and TMI Modifications.

> INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

Forecast	Achieved

8007010390

## AVERAGE DAILY UNIT POWER LEVEL

50-31/	
Calvert Cliffs	#1
6/12/80	
S.D.Merson	
301-234-5240	
	Calvert Cliffs 6/12/80 S.D.Merson

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY 17	AVERAGE DAILY POWER LEVEL (MWe-Net) 788
	18	831
-	19	825
24	20	617
723	21	114
767	22	808
772	23	822
785	24	824
820	25	791
830	. 26	819
789	27	821
822	28	823
798	29	823
830	30	823
827	31	820
821		

### INSTRUCTIONS

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to many .

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

## UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-317 UNIT NAME Cliffs #1 DATE 6/12780 COMPLETED BY S.D.Merson TELEPHONE 301-234-5240

## REPORT MONTH May, 1980

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor 3	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
80-08	800430	F	91.2 23.2	A	4	LER 80-24/3L LER 80-27/1T	СВ	<b>РИМРХХ</b> НТЕТСН	Leak on #11B Reactor coolant pump control bleed-off line. Plant was already off for outage (80-07). Leak in the after-cooler on #12 instrument air compressor.
F: For S: Sche		B-Mai C-Ref D-Reg E-Ope F-Adr G-Ope	n: ipment Fai ntenance of ueling ulatory Re- rator Train ninistrative grational Er- ter (Explain	r Test striction ing & Li ror (Exp	icense Exa	3 mination	3-Auto	ual ual Scram. muatic Scram. tinuation d reduction	4 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit 1 - Same Source

6/6/80

### REFUELING INFORMATION REQUEST

- 1. Name of Facility: Calvert Cliffs Nuclear Power Plant, Unit No. 1
- 2. Scheduled date for next Refueling Shutdown: October 17, 1980
- 3. Scheduled date for restart following refueling: December 10, 1980
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

Resumption of operation after refueling will require changes to Technical Specifications. The changes will be such as to allow operation of the plant with a fresh reload batch and reshuffled core.

 Scheduled date(s) for submitting proposed licensing action and supporting information.

September 11, 1980

6. Important licensing considerations associated with the refueling.

Reload fuel will be similar to that reload fuel inserted into the previous cycle.

- The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
  - (a) 217 (b) 364

Spent Fuel Pools are common to Units 1 and 2.

 The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

> 1056 Licensed 728 Currently Installed 774 Licensed Addition is Planned

The projected date of the last refueling that can be discharged to the Spent Fuel Pool assuming the present licensed capacity.

October, 1983

# SUMMARY OF UNIT 1 OPERATING EXPERIENCE - MAY 1980

- 5/1 At the beginning of this reporting period, Unit 1 was shutdown for repair of the Reactor Coolant System leak from 11B Reactor Coolant Pump control bleed off line.
- 5/3 Commenced Reactor Coolant System fill at 0250. At 1830, heatup of the Reactor Coolant System was begun.
- 5/4 The Reactor was brought critical at 1635 and the unit paralleled at 1910.
- 5/5 At 1020, load was limited to 765 MWe to investigate saltwater leakage into the main condenser.
- 5/8 After plugging 1 condenser tube load was increased to capacity (850 MWe) at 0030. At 1900 load was decreased to 805 MWe to investigate saltwater leakage into the main condenser.
- 5/9 Resumed full load operation (870 MWe) at 0945 after plugging one condenser tube.
- 5/11 Load was reduced to 710 MWe at 1445 to investigate saltwater leakage into the main condenser.
- 5/12 One condenser tube was plugged and load was increased to capacity (865 MWe) at 0300.
- 5/13 At 0900 decreased load to 780 We to investigate saltwater leakage into the main condenser.

- 5/17 At 1035 load was reduced to 750 MWe for Main Turbine Control Valve Testing. Resumed full load operation (860 MWe) at 1435.
- 5/20 The unit was manually tripped at 1803 when the service water system became airbourd. The cause was a leak in the aftercooler on No. 12 Instrument Air Compressor.
- 5/21 The Reactor was brought critical at 1320 and the unit paralleled at 1714.
- 5/22 At 0500 full load operation (860 MWe) was resumed.
- 5/25 Load was decreased to 760 MWe at 1500 to investigate saltwater leakage into the main condenser. At 2230, after indications of leakage disappeared, load was increased to 860 MWe.
- 5/31 At the end of this reporting period, Unit 1 was operating at 860 MWe with the Reactor at 100% power.

# SAFETY-RELATED MAINTENANCE

UNIT	ONE	
GROUP	I&C	
MONTH _	MAY	YEAR _ 1980

MR NO DATE	CAUSE	RESULT	CORRECTIVE ACTION
0-80-870 3-17-80	Defective isolation module in sensor channel ZE.	Test light would not reset.	Replaced the isolation module.
IC-80-045 3-25-80	Defective power supply and pre-amplifier.	Channel 'A' Wide Range Nuclear Instrument would not calibrate.	Replaced the power supply and the preamp?'fier.
IC-80-038 2-26-80	Defective Signal Isolator 1-E/E-5313D	Containment Pressure Instrument out of tolerance.	Replaced the defective signal isolator 1-E/E-5313D
0-80-808 3-11-80	Dirty terminals on relay XK45 in the BR Relay Cabinet.	1-SU-6529 and 1-SU-6531 would not function.	Cleaned the terminals on the Relay XK45.
	0-80-870 3-17-80 IC-80-045 3-25-80 IC-80-038 2-26-80 0-80-808	MR NO DATECAUSE0-80-870 3-17-80Defective isolation module in sensor channel ZE.IC-80-045 3-25-80Defective power supply and pre-amplifier.IC-80-038 2-26-80Defective Signal Isolator 1-E/E-5313D0-80-808 3-11-80Dirty terminals on relay XK45 in the	0-80-870 3-17-80Defective isolation module in sensor channel ZE.Test light would not reset.IC-80-045 3-25-80Defective power supply and pre-amplifier.Channel 'A' Wide Range Nuclear Instrument would not calibrate.IC-80-038 2-26-80Defective Signal Isolator 1-E/E-5313DContainment Pressure Instrument out of tolerance.0-80-808 3-11-80Dirty terminals on relay XK45 in the1-SU-6529 and 1-SU-6531 would not function.

SAFETY-RELATED MAINTENANCE

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Machine Shop			
Мау		YEAR -	<b>1</b> 19
	Machine Shop May		· • • •

MALFUNCTION LER OR SYSTEM OR COMPONENT MR NO. - DATE OUTAGE CAUSE RESULT CORRECTIVE ACTION Pilot and main valve 1-ERV-402 M-80-3026 Seat leakage. Pilot valve disc 2/12/80 Pressurizer Relief seating surfaces were was replaced and lapped in. Main Valve cut. 个 建建设合 注 disc was lapped in. . . . . . . . . 1-ERV-404 0-79-4412 Pilot and main valve Excessive seat Pilot and main valve Pressurizer Relief 2/12/80 seating surfaces were leakage. discs were replaced Valve cut. and lapped in. . 1 . ..