



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

JUN 17 1980

Docket No. 50-364

Mr. F. L. Clayton, Jr.  
Senior Vice President  
Alabama Power Company  
600 North 18th Street  
Birmingham, Alabama 35291

Dear Mr. Clayton:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR FARLEY 2 OPERATING LICENSE  
APPLICATION

As a result of our continuing review of the operating license application for the Joseph M. Farley Nuclear Plant Unit 2, we have developed the enclosed request for additional information. This information was discussed in a telephone conversation with your representatives on May 23, 1980.

Please provide the information requested in the enclosure. Our review schedule is based on the assumption that the additional information will be available for our review by June 27, 1980. If you cannot meet this date, please inform us within 7 days after receipt of this letter so that we may revise our scheduling.

Sincerely,

A. Schwencer, Chief  
Licensing Branch No. 2  
Division of Licensing

Enclosure:  
Request for Additional  
Information

cc: See next page

Mr. F. L. Clayton, Jr., Senior Vice  
President  
Alabama Power Company  
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Birmingham, Alabama 35291

cc: Mr. Alan R. Barton  
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Mr. Ruble A. Thomas  
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Mr. George F. Trowbridge  
Shaw, Pittman, Potts and Trowbridge  
1800 M Street, N. W.  
Washington, D. C. 20036

ENCLOSURE

REQUESTS FOR ADDITIONAL INFORMATION  
JOSEPH M. FARLEY NUCLEAR PLANT UNIT 2  
DOCKET NO. 50-364

Requests from the following branch in NRC are included in this enclosure. Requests and pages are numbered sequentially with respect to requests transmitted following issuance of SER Supplement No. 3.

<u>Branch</u>	<u>Page No.</u>
Materials Engineering	121.6 121.7

121.0 MATERIALS ENGINEERING BRANCH - INSERVICE INSPECTION SECTION

121.2 Your response to Question 121.1 does not provide sufficient information to complete our evaluation regarding your preservice inspection program. For each component which cannot be completely examined in accordance with ASME Code Section XI requirements, submit a relief request in accordance with the guidelines in Appendix A of Question 121.1.

Specific information should include the extent of the Section XI requirements that were performed, alternative nondestructive examinations, and description of the measures required to perform the required examinations. In the case where relief is requested for piping system welds, provide a list of the welds that cannot be completely examined to Section XI requirements.

Where construction code examinations are substituted for preservice examination requirements, in accordance with IWB-2100b and IWC-2100b in Section XI, identify the specific construction code NDE method.

Identify the specific parts of the ECCS and RHRS where IWC-1220(a) and (c) exemptions are used and a preservice examination was not performed.

121.3 For the preservice inspection program, the following information necessary to clarify specific aspects of the FSAR and additional information supplied in your letter of April 21, 1980:

- (1) A list of all welds, the percentage of volume which can be examined, and alternate examinations to be performed for which Notes 9 and 10 in Table 5.2-33 and Notes 5 and 6 in Table 5.2-34 apply.
- (2) The type and extent of examination of the nozzle inner radii region of the reactor vessel, steam generator, and pressurizer, and the nozzle to safe end welds in the steam generator.
- (3) A list of all welds in branch pipe connections exceeding 6 inches in diameter for which relief is requested. A detailed description or drawing depicting inaccessibility and estimates of the extent of examination as a percentage of total weld volume are required.
- (4) Exclusion sizes in the piping pressure boundary you have used based on IWB-1220b of Section XI. Support these sizes with documentation if they are not otherwise exempted.
- (5) A list of all Class 2 welds which cannot be examined volumetrically due to thin wall thickness.
- (6) A relief request for the charging pump casing welds.
- (7) A discussion of the impact on plant safety of the omission of NDE on nozzle to vessel welds of the residual heat exchanger, as

referenced in Note 3, Table 5.2-34 of the FSAR, and of pressurizer safety welds 7, 9, 10, and 11 in additional information of your April 21, 1980 letter.

- (8) The percentage of weld volumes in the regenerative heat exchanger which can be examined volumetrically.