UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA ST., N.W., SUITE 3100
ATLANTA, GEORGIA 30303

Report Nos. 50-269/80-16, 50-270/80-11 and 50-287/80-10

Licensee: Duke Power Company
422 South Church Street
Charlotte, NC 28242

Facility Name: Oconee

Docket Nos. 50-269, 50-270 and 50-287

License Nos. DPR-38, DPR-47 and DPR-55

Inspection at Oconee site near Seneca, South Carolina

Inspected by: E H Brooks — E. H. Brooks — 5/14/80 Date Signed

Approved by: C M Upright — C. M. Upright, Acting Section Chief, RONS Branch — 5/16/80 Date Signed

SUMMARY

Inspection on April 22-25, 1980.

Areas Inspected

This routine, announced inspection involved 26 inspector-hours on site in the areas of containment integrated leakage rate testing which included inspection of test procedures and potential problem areas during the Unit 2 leakage rate test, and inspection of the plant surveillance program of pipe support and restraint systems for Units 1, 2 and 3.

Results

Of the two areas inspected, no items of noncompliance or deviations were identified.

DETAILS

1. Persons Contacted

Licensee Employees

*J. E. Smith, Manager
*J. M. Davis, Superintendent of Maintenance
*L. Wilke, Engineer, Maintenance
B. Carney, Assistant Engineer, Maintenance
W. Neuman, Test Engineer, Performance
*R. Bond, Project License Engineer
J. Snyder, Engineer, Corporate Office
K. Wilson, Engineer, Corporate Office
G. Davenport, Performance Engineer
J. Collier, Test Engineer, Performance

NRC Resident Inspectors

F. Jape
*W. Orders
D. Myers

*Attended exit interview

2. Exit Interview

The inspection scope and findings were summarized on April 25, 1980 with those persons indicated in paragraph 1, above.

3. Licensee Action on Previous Inspection Findings

a. (Closed) Unresolved Item (269/270-287/80-06-02); Review Duke system design to identify those systems which may rupture during the design basis accident and include these systems in determining the integrated leakage rate. The licensee has completed the system review and additional Type "C" testing required on Unit 1 to determine the integrated leakage rate. Duke procedure TT/1/A/151/2, Supplemental Type "C" Leak Rate Test, summarizes the test results. Summation of the measured integrated leakage and the supplementary Type "C" leakage resulted in a total containment leakage rate of 0.050496 weight percent per day which is less than the maximum allowable leakage rate (0.75 Lt) of 0.0581 weight percent per day. Test data and documentation of all supplementary tests performed will be included in the containment integrated leakage rate test report to be submitted to the NRC.

b. (Open) Noncompliance-Infraction (269/79-15-01) - Two snubbers were found by the inspector with fluid ports in the reservoirs uncovered and one loose locknut on a cylinder extension tie rod. These findings were subsequent to the licensee's inspection and constitute noncompliance to technical specifications which require adequate inspection

to verify snubber operability. The licensee's response to the notice of violation stated in effect that the suppressor removal/installation procedure would be revised to clarify correct installation of suppressors and that a formal inspection training program was in preparation. The inspector reviewed the revised removal/installation procedure MP/0/A/3000/24 dated 3-25-80. However, a formal training program for inspection of snubbers has not to date been instituted by the licensee. The licensee stated that the formal training program will be initiated by July 1, 1980.

This item remains open pending completion of the inspection training program.

4. Unresolved Items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of noncompliance, or deviations. An unresolved item disclosed during the inspection is discussed in Paragraph 6.

5. Primary Containment Integrated Leakage Rate Test

The inspector met with the licensee to discuss supplementary type "C" leak testing conducted on Unit 1 and revised procedures for leak testing the unit 2 reactor building. Procedure TT/1/A/151/2, Supplemental Type "C" Leak Rate Test, was performed February 6-11, 1980 and the procedure completion was approved on March 3, 1980. Test results are discussed in Paragraph 3.a. of this report. This closes inspector followup item (269/270/287/80-06-04).

The licensee had previously committed to revising the valve alignment checklist in the reactor building Integrated Leak Rate Test Procedure PT/0/A/150/3 prior to Unit 2 containment integrated leak rate testing (ILRT) since it was not adequate to assure proper venting and draing of systems during the ILRT. A copy of revised PT/0/A/150/3 including a revised valve lineup was provided to the inspector for Region II review. Pending review of the revised procedure inspector followup item (269/270/287/80-06-03) remains open.

6. Inspection of Pipe Support and Restraint Systems

April 23, the inspector accompanied by the NRC resident inspector and licensee personnel conducted an inspection of accessible and inaccessible safety-related hydraulic suppressors (snubbers) installed in Unit 2 of the Oconee Nuclear Station. Approximately 90 per cent of the snubbers inside the reactor building (inaccessible) and 50 percent of the snubbers outside the reactor building (accessible) were visually inspected for operability as required by the Oconee technical specifications. One accessible main steam line snubber (2-01A-0-1401A-R14) was found to have a "kinked" supply tube from the reservoir to the snubber. Operability of this snubber was considered questionable by the inspector and the licensee agreed to perform additional inspection to confirm operability. This matter is identified for later inspection as inspector followup item (50-270/80-11-01).

In early March 1980, the licensee completed inspection of Unit 2 snubbers and found three with completely empty reservoirs. One reservoir was refilled and declared to be inoperative. The licensee planned to functionally test the remaining two snubbers with empty reservoirs to determine their operability. The licensee was advised at the exit interview that it is the NRC position that empty reservoirs clearly constitute inoperable snubbers and accordingly the inspection schedule for Unit 2 snubbers should be changed in accordance with technical specifications to four months on the basis of three inoperable snubbers. The licensee was advised that Region II would confirm the NRC position on this matter and would advise accordingly. The NRC position as stated above was confirmed and this information was relayed to the licensee by the Oconee Resident Inspector on April 29, 1980. The licensee does not concur with the NRC position. In accordance with the Oconee plant technical specification plant operation with snubbers in a degraded condition is a reportable occurrence. The licensee will report the occurrence as required. The matter of snubber operability as discussed above is identified as an unresolved item (50-270/80-11-02).

During a previous inspection (report number 50-269/79-15) eleven snubbers installed in Unit 1 were found by the licensee to be incorrectly oriented due to loose locking nuts between the cylinders and rod ends. The licensee has proceeded with corrective measures involving addition of a set screw in the locking nut. All snubbers visually inspected in Unit 2 had been modified to incorporate the set screw. This closes open item (269/79-15-02).