Docket File

APR 3 0 1973

Docket No. 50-312

Mr. E. K. Davis, General Manager
Sacramento Municipal Utility District
6201 S Street, P. O. Box 15830
Sacramento, California 95813

Dear Mr. Davis:

To assure that adequate protection of the core is being provided during a postulated loss of coolant accident (LOCA) at the Rancho Seco Nuclear Generating Station, it is requested that you provide a reanalysis of the following items:

(1) peak core cladding temperatures for the worst case large reactor coolant cold leg break using the REFLOOD code modified to include the carryover rate function to predict entrainment,

(2) peak core cladding temperature for the worst case small reactor coolant pipe break, and

(3) peak core cladding temperature for a double-ended break in the line that connects a core flooding tank (CFT) to the reactor vessel using the conservative thermal and hydraulic assumptions developed for the Oconee analysis. The location of this break should be between the reactor vessel nozzle and the first check valve leading to the CFT.

It is requested that you expedite your response to minimize impact on the licensing schedule review.

Sincerely,

Original Signed by
Albert Schwencer

A. Schwencer, Chief
Pressurized Water Reactors
Branch No. 4
Directorate of Licensing

cc: See next page

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cc: Mr. David S. Kaplan, Secretary
and Attorney
6201 S Street, P. O. Box 15839
Sacramento, California 95813

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