

B 06/07/78

REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)
DISTRIBUTION FOR INCOMING MATERIAL 50-346

REC: STOLZ J F
NRC

ORG: ROE L E
TOLEDO EDISON

DOC DATE: 06/02/78
DATE RCV: 06/06/78

DOCTYPE: LETTER NOTARIZED: NO COPIES RECEIVED
SUBJECT: LTR 1 ENCL 3
FORWARDING REVISED PAGE 1, 5, 10, 13, 14 AND 15 TO ATTACHMENT 1 OF
APPLICANT'S LTR DTD 05/26/78 PERTAINING TO THE DETERMINATION OF TOTAL RC
FLOWRATE AND ITS ACCURACY FOR UNIT 1 AT 100% PWR LEVEL.

PLANT NAME: DAVIS BESSE - UNIT 1

REVIEWER INITIAL: XJM
DISTRIBUTOR INITIAL: M

***** DISTRIBUTION OF THIS MATERIAL IS AS FOLLOWS *****

NOTES:

1. SEND ALL AMENDMENTS TO J. ROE

PSAR/FSAR AMDTS AND RELATED CORRESPONDENCE
(DISTRIBUTION CODE B001)

FOR ACTION: ASST DIR VASSALLO**LTR ONLY BR CHIEF LWR#1 BC**LTR ONLY
PROJ MGR ENGLE**W/ENCL LIC ASST HYLTON**LTR ONLY

INTERNAL:	REG FILE**W/ENCL	NRC PDR**W/ENCL
	I & E**W/2 ENCL	OELD**LTR ONLY
	OPERATOR LIC BR**W/ENCL	EMERGENCY PLAN BR**W/ENCL
	QAB**W/ENCL	CASE**LTR ONLY
	MIPC**LTR ONLY	AD FOR ENG**LTR ONLY
	MECH ENG BR**W/ENCL	STRUCTURAL ENG BR**W/ENCL
	MATERIAL ENG BR**W/2 ENCL	AD FOR REAC SFTY**LTR ONLY
	REACTOR SYSTEMS BR**W/ENCL	ANALYSIS BR**W/ENCL
	CORE PERFORMANCE BR**W/ENCL	AD FOR PLANT SYSTEMS**LTR ONLY
	AUXILIARY SYS BR**W/ENCL	CONTAINMENT SYSTEMS**W/ENCL
	I & C SYSTEMS BR**W/ENCL	POWER SYS BR**W/ENCL
	AD FOR SITE TECH**W/4 ENCL	AD FOR SITE ANLYS**LTR ONLY
	ACCIDENT ANALYSIS**W/ENCL	EFFLUENT TREAT SYS**W/ENCL
	RAD ASSESSMENT BR**W/ENCL	KIRKWOOD**W/ENCL

EXTERNAL:	LPDR'S	HANAUER W/ENCL
	PT. CLINTON, OH**W/ENCL	STELLO W/ENCL
	TIC**W/ENCL	ENGINEERING BR W/ENCL
	NSIC**W/ENCL	REACTOR SAFETY BR W/ENCL
	ACRS CAT B**W/16 ENCL	PLANT SYS BR W/ENCL
		EEB W/ENCL

DISTRIBUTION: LTR ⁶²~~56~~ ENCL ⁵²~~46~~
SIZE: 1P+6P

CONTROL NBR: 781580007
MA 4
BT

***** THE END *****

8001230646



LOWELL E. ROE
Vice President
Facilities Development
(419) 259-5242

Docket No. 50-346

License No. NPF-3

June 2, 1978

Serial No. 440

Director of Nuclear Reactor Regulation
Attention: Mr. John F. Stolz, Chief
Light Water Reactors Branch No. 1
Division of Project Management
United States Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Stolz:

We are supplying revised Pages 1, 5, 10, 13, 14, and 15 for attachment 1 of our letter to you dated May 26, 1978 (Serial No. 436). The revisions produce a small increase in the reactor coolant flow rate that is calculated from the heat balance, but have no significant effect on the flow rate uncertainty calculation.

Yours very truly,

Enclosures

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DETERMINATION OF TOTAL RC FLOWRATE AND
ITS ACCURACY FOR DAVIS-BESSE 1 AT 100%
POWER LEVEL

INTRODUCTION

In a B&W nuclear power plant, Gentile flowmeters are used to measure Loop 1 and 2 reactor coolant flowrates. These primary loop flowmeters are not calibrated prior to installation. Loop 1 and 2 feedwater flowrates are measured with calibrated flow meters and B&W utilizes a plant heat balance to set the calibration of the primary loop flowmeters.

An error analysis on the equations used to determine the total reactor core flowrate (Loop 1 plus Loop 2) has revealed that the errors in reactor coolant temperatures and feedwater flowmeter differential pressure are the most significant terms in calculating accurate values of total reactor core flowrate.

SUMMARY

The calculated RC flowrate for Davis-Besse 1 at 100% power is 113.5% times the design flowrate of 352,000 gpm. The accuracy is $\pm 2.2\%$. This was determined with RC temperature instrument string errors equal to $\pm 0.79\text{F}$ and feedwater flowmeter ΔP errors equal to $\pm 1.25\%$ and steam temperature errors equal to $\pm 4.2\text{F}$.

Reactor coolant system temperatures are measured with $\pm 1/4\%$ accurate pre-calibrated RTD's over a range of 520 to 620F. Similarly, the Bailey Meter Company differential pressure transmitters are calibrated to $\pm 1/2\%$ at time of installation. The two feedwater flowmeters are calibrated to $\pm 1/2\%$ prior to installation.

For normal everyday conditions in the instrumentation area of the plant, B&W has determined the accuracy of all input measurements used in this error analysis. (Refer to page 4.)

COMPARISON OF HOT LEG TEMPERATURES (T_H)

FOR DAVIS-BESSE 1 AT 100% POWER ON

APRIL 5, 1978

<u>TIME</u>	<u>LOOP 1 T719 ($^{\circ}$F)</u>	<u>LOOP 1 T720 ($^{\circ}$F)</u>
14:39	605.6	605.8
14:44	606.0	606.1
14:50	606.0	606.2
14:54	605.9	606.1
14:59	605.6	606.1
15:04	605.8	606.1
15:09	605.9	605.9
15:14	605.9	606.1
Midpoint	605.8	606.0
Span	± 0.2	± 0.2

Average $T_H = 605.9^{\circ}$ F during 35 minutes

<u>TIME</u>	<u>LOOP 2 T729 ($^{\circ}$F)</u>	<u>LOOP 2 T728 ($^{\circ}$F)</u>
14:39	604.9	604.3
14:44	605.3	604.8
14:50	605.4	604.9
14:54	605.1	604.4
14:59	605.1	604.6
15:04	605.1	604.4
15:09	605.0	604.6
15:14	605.2	604.7
Midpoint	605.15	604.6
Span	± 0.25	± 0.3

Average $T_H = 604.88^{\circ}$ F during 35 minutes

PRIMARY SIDE ENTHALPY CALCULATION:

NOMENCLATURE

H_H = Reactor Coolant hot leg enthalpy

H_C = Reactor Coolant cold leg enthalpy

H_S = Enthalpy of steam at steam generator outlet

H_F = Enthalpy of feedwater to the steam generator

ΔH = Change in enthalpy, an added subscript: 'pri' or 'sec' implies the primary and secondary loops respectively.

LOOP 1:

H_H @ 605.9°F and 2159 psia = 622.26 Btu/lb

H_C @ 558.7°F and 2220 psia = 558.14 Btu/lb

ΔH_{pri} = 64.12 Btu/lb

LOOP 2:

H_H @ 604.9°F and 2159 psia = 620.79 Btu/lb

H_C @ 558.9°F and 2220 psia = 558.39 Btu/lb

ΔH_{pri} = 62.40 Btu/lb

SECONDARY SIDE ENTHALPY CALCULATION:

LOOP 1:

H_S @ 595.5°F and 920 psia = 1254.63 Btu/lb

H_F @ 460°F and 958 psia = 441.73 Btu/lb

ΔH_{sec} = 812.90 Btu/lb

LOOP 2:

H_S @ 596.5°F and 897 psia = 1258.03 Btu/lb

H_F @ 460°F and 972 psia = 441.74 Btu/lb

ΔH_{sec} = 816.29 Btu/lb

$$\text{or } W_{RE1} (\Delta H_1)_{\text{PRIMARY}} = (W_F \Delta H_{\text{sec}})_{\text{LOOP1}} - \frac{74.76 \times 10^6}{2} \frac{\text{Btu}}{\text{hr}}$$

$$\text{and } W_{RE2} (\Delta H_2)_{\text{PRIMARY}} = (W_F \Delta H_{\text{sec}})_{\text{LOOP2}} - \frac{74.76 \times 10^6}{2} \frac{\text{Btu}}{\text{hr}}$$

Solving for W_{RE1} and W_{RE2} :

$$W_{RE1} = \frac{(W_F \Delta H_{\text{sec}})_{\text{LOOP1}} - 37.38 \times 10^6 \text{ Btu/hr}}{(\Delta H_1)_{\text{PRIMARY}}}$$

$$\text{and } W_{RE2} = \frac{(W_F \Delta H_{\text{sec}})_{\text{LOOP2}} - 37.38 \times 10^6 \text{ Btu/hr}}{(\Delta H_2)_{\text{PRIMARY}}}$$

From page 10

$$(\Delta H_1)_{\text{PRI}} = 64.12 \text{ Btu/lb}$$

$$(\Delta H_2)_{\text{PRI}} = 62.40 \text{ Btu/lb}$$

$$(\Delta H_{\text{sec}})_{\text{LOOP1}} = 812.90 \text{ Btu/lb}$$

$$(\Delta H_{\text{sec}})_{\text{LOOP2}} = 816.29 \text{ Btu/lb}$$

Substituting the above values and noting that

$$W_{F\text{LOOP1}} = 5.812 \times 10^6 \text{ lb/hr}$$

$$W_{F\text{LOOP2}} = 5.785 \times 10^6 \text{ lb/hr}$$

$$W_{RC1} = \frac{(5.812 \times 10^6)(812.90) - 37.38 \times 10^6 \text{ Btu/hr}}{(64.12)}$$

$$= 73.10 \times 10^6 \text{ lb/hr}$$

$$W_{RC2} = \frac{(5.785 \times 10^6)(816.29) - 37.38 \times 10^6 \text{ Btu/hr}}{(62.40)}$$

$$= 75.08 \times 10^6 \text{ lb/hr}$$

$$\text{Total RC flow rate} = W_{RET} = W_{RC1} + W_{RC2}$$

$$= 148.18 \times 10^6 \text{ lb/hr}$$

The total RC flowrate in gpm may be calculated as

$$W_{RET}(\text{gpm}) = \frac{W_{RET}(\text{lb/hr})}{\frac{60}{7.4805} \times \rho_{\text{cold leg}}}$$

$$\rho_{\text{cold leg}} = 46.26 \text{ lb/ft}^3 \text{ at } 558.8^\circ\text{F and } 2220 \text{ psia}$$

$$W_{RET}(\text{gpm}) = 3.994 \times 10^5 \text{ gpm}$$

$$\text{The design flowrate for RC system} = 4 \times 88000 \text{ gpm}_{\text{pump}}$$

$$= 352,000 \text{ gpm}$$

$$\text{Therefore the ratio to design flowrate is: } \frac{3.994 \times 10^5}{3.52 \times 10^5}$$

$$= 113.5\%$$

ERROR ANALYSIS OF CALCULATED RC FLOWRATE

The basic equation for Loop 1 or 2 RC flowrate is:

$$W_{RC} = \frac{W_F \left[\frac{\Delta H}{\text{sec}} \right] - \frac{C}{2}}{[\Delta H_{\text{per I]}}$$

where $C = Q_{\text{pumps}} - Q_{\text{rad. losses}} = 74.76 \times 10^6 \text{ Btu/hr}$

If one assumes an extreme case of a 10% error in C, then C would be equal to $8.2236 \times 10^7 \text{ Btu/hr}$ rather than $74.76 \times 10^6 \text{ Btu/hr}$.

Then total RC Flowrate would become:

$$W_{RC1} = \frac{5.812 \times 10^6 \times 812.90 - 41.12 \times 10^6 \text{ Btu/hr}}{64.12} = 73.04 \times 10^6 \text{ lb/hr}$$

$$W_{RC2} = \frac{5.785 \times 10^6 \times 816.29 - 41.12 \times 10^6 \text{ Btu/hr}}{62.40} = 75.02 \times 10^6 \text{ lb/hr}$$

Total RC flow rate = $148.06 \times 10^6 \text{ lb/hr}$ and the

Change in total RC flowrate is 0.08%.

For this small change we shall assume C is a constant and is $74.76 \times 10^6 \text{ Btu/hr}$. If it is in error, its influence on the final value is insignificant.

The small corrections to the H_H values (as given on revised p.10) were not made in the error analysis on subsequent pages, because the corrections have a negligible effect on the error calculations.