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Albert Schwencer, Chief, Operating Reactors Branch #1, DOR

FORTHCOMING MEETING WITH DUKE POWER COMPANY (OCONEE NUCLEAR STATION)

Date and Time: July 8, 1976, Thursday
9:45 a.m.

Location: Phillips Building
Room 316-B

Purpose: To discuss causes and prevention of
PWR reactor vessel overpressurization
events. (See attached agenda)

Participants: Duke Power Company

M. Tuckman, et al

NRC

C. Berlinger
R. Fluegge
R. Gamble
C. G. Zech, et al

Original signed by

Gary G. Zech, Project Manager
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
Agenda

cc w/encl:
See next page

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|-----------|------------|--|--|--|--|--|
| OFFICE → | | | | | | |
| SURNAME → | FOR: ORB#1 | | | | | |
| DATE → | GGZech: Ib | | | | | |
| | 7/7/76 | | | | | |

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cc: NRC PDR
Local PDR
B. Rusche
E. Case
R. Boyd
R. Heineman
V. Stello
H. Denton
D. Skovholt
R. Denise
R. DeYoung
R. Maccary
D. Ross
R. Tedesco
K. Goller
D. Eisenhut
V. Moore
R. Vollmer
M. Ernst
W. Gammill
P. Collins
C. Heltemes
R. Houston
T. Speis
R. Clark
J. Stolz
K. Kniel
O. Parr
W. Butler
D. Vassallo
J. Knight
S. Pawlicki
I. Sihweil
P. Check
T. Novak
Z. Rosztoczy
V. Benaroya
G. Lainas
T. Ippolito
D. Ziemann
G. Lear
R. Reid
L. Shao
R. Baer
A. Schwencer
B. Grimes

G. Knighton
B. Youngblood
W. Regan
D. Bunch
J. Collins
W. Kreger
R. Ballard
M. Spangler
J. Stepp
L. Hulman
T. Carter
OR PM
S. Sheppard
R. Fraley, ACRS(16)
SD(3)
OI&E(3)
OELD
B. Faulkenberry, OI&E
Receptionist
Principal Staff Participants

bcc: T. B. Abernathy, TIC
J. R. Buchanan, NSIC

Dudley Thompson, Acting Director
Division of Field Operations
Office of Inspection and Enforcement
Mail Stop 359 E/W

Agenda for Meeting to Discuss
Reactor Vessel Overpressurization Events

STAFF PRESENTATION

- Statement of Problem
- Significance of Problem
- Definition/Statement of Acceptance Criteria
- Long Term Fix
 - Design Modifications to Prevent Overpressure Transients
 - Mitigate Consequences
- Interim Fix
 - Reduction of Likelihood
 - Mitigate Consequences

LICENSEE PRESENTATION

- Past History
 - Identify Overpressure Incidents
 - Evaluate Susceptibility of Plant to Overpressure of RCS
 - Quantify Occasions of Water Solid and/or Cold Shutdown Operation
- Design/Procedural Modifications to Prevent Reoccurrence
 - Discuss Success/Failures of Fixes
- Pressurizer Relief Valve Performance Parameters
 - Valve Cycle Times
 - Control Logic
 - Range/Accuracy of Wide Range Pressure Sensors

LICENSEE PRESENTATION (continued)

- Pressurizer Relief Valve Performance Parameters (cont)
 - Display Instrumentation and Alarms
- Assessment of Possibility of Operating Plant Without Being Water Solid
- Discuss Capability of Providing and Controlling a Nitrogen Blanket

GENERAL DISCUSSION

- Interim and Long Term Fixes

Information to be Provided to

NRC at the Meeting

- 4 copies of large sized P&ID's charging, letdown, RHR and reactor coolant systems
- Viewgraphs of System P&ID's for use in discussion
- 4 copies of operating procedures for operations during cold shutdown and water solid conditions
- Fluence Data
 - Capsules removed
 - Removal schedule for future
 - Copy of NSSS Surveillance Program
 - Data on hand not yet submitted