NRC STRIBUTION FOR PART 50 DOC T MATERIAL
(TEMPORARY FORM)

CONTROL NO: 5965

FILE:

FROM:	DATE OF DOC	DATE REC'D	LTR	TWX	RPT	OTHER
Duke Power Co Charlotte, N.C. A.C. Thies	5-30-75	6-2-75	XX[X]			

TO:	ORIG	CC	OTHER	SENT AEC PDR	SENT LOCAL PDR
A. Giambusso	1 Signed			XXXX	XXXX

CLASS	UNCLASS	PROP INFO	INPUT	NO CYS REC'D	DOCKET NO:
	XXXX			1	50-269/270/287

DESCRIPTION:
Ltr. ref their transmittal of 4-16-75 which was in response to our 3-14-75 ltr....providing add'l info.concerning results of a review of Oconee Nuclear Station system capabilities & oper. procedures conducted to evaluate possibility of significant changes in chemical concentrations during long ter after a postulated loss-of-coolant accid. .....(LOCA)

PLANT NAME: Oconee 1-2-3

ENCLOSURES:

FOR ACTION/INFORMATION VCR 6-4-75

BUTLER (L) W/ Copies	SCHWENCER (L) W/ Copies	ZIEMANN (L) W/ Copies	REGAN (E) W/ Copies
CLARK (L) W/ Copies	STOLZ (L) W/ Copies	DICKER (E) W/ Copies	LEAR (L) W/ Copies
PARR (L) W/ Copies	VASSALLO (L) W/ Copies	KNIGHTON (E) W/ Copies	SPETS W/ Copies
KNIEL (L) W/ Copies	PURPLE (L) W/6 Copies	YOUNGBLOOD (E) W/ Copies	LICENSING PROJECT MANAGER W/ Copies

INTERNAL DISTRIBUTION

REG FILE	TECH REVIEW	DENTON	LIC ASST.	A/T IND
NRC PDR	SCHROEDER	GRIMES	R. DIGGS (L)	BRAITMAN
OGC, ROOM P-506A	MACCARY	GAMMILL	H. GEARIN (L)	SALTZMAN
GOSSICK/STAFF	KNIGHT	KASTNER	E. GOULBOURNE (L)	MELTZ
CASE	PAWLICKI	BALLARD	P. KREUTZER (E)	
GIAMBUSSO	SHAO	SPANGLER	J. LEE (L)	PLANS
BOYD	STELLO		M. MAIGRET (L)	MCDONALD
MOORE (L)	HOUSTON	ENVIRO	S. REED (E)	CHAPMAN
DEYOUNG (L)	NOVAK	MULLER	W. SERVICE (L)	DUBE (Ltr)
SKOVHOLT (L)	ROSS	DICKER	S. SHEPPARD (L)	E. COUPE
GOLLER (L) (Ltr)	IPPOLITO	KNIGHTON	M. SLATER (E)	PETERSON
P. COLLINS	TEDESCO	YOUNGBLOOD	H. SMITH (L)	HARTFIELD (2)
DENISE	J. COLLINS	REGAN	S. TEETS (L)	KLECKER
REG OPR	LAINAS	PROJECT LDR	G. WILLIAMS (E)	EISENHUT
FILE & REGION (2)	BENAROYA	Mann	V. WILSON (L)	WIGGINTON
MPIC	VOLLMER	HARLESS	R. INGRAM (L)	Varga
STEELE				

EXTERNAL DISTRIBUTION

1 – LOCAL PDR Walhalla, S.C.
1 – TIC (ABERNATHY) (1)(2)(10) – NATIONAL LABS
1 – NSIC (BUCHANAN) 1 – W. PENNINGTON, Rm E-201 GT
1 – ASLB 1 – CONSULTANTS
1 – Newton Anderson NEWMARK/BLUME/AGBABIAN
14 ACRS HOLDING/SENT Sheppard

1 – PDR-SAN/LA/NY
1 – BROOKHAVEN NAT LAB
1 – G. ULRIKSON, ORNL
1 – AGMED (RUTH GUSSMAN) RM B-127 GT
1 – J. D. RONEY LES, Rm E-201 GT

7912190825

DUKE POWER COMPANY
POWER BUILDING
422 SOUTH CHURCH STREET, CHARLOTTE, N. C. 28201

A. C. THIES
SENIOR VICE PRESIDENT
PRODUCTION AND TRANSMISSION

P. O. Box 2178

2 VIR

May 30, 1975

Mr. Angelo Giambusso, Director
Division of Reactor Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Re: Oconee Nuclear Station
Docket Nos. 50-269, -270, -287

Dear Mr. Giambusso:

The purpose of this letter is to provide additional information in accordance with my transmittal of April 16, 1975. That transmittal was in response to a March 14, 1975 letter from Mr. R. A. Purple and described the results of a review of Oconee Nuclear Station system capabilities and operating procedures conducted to evaluate the possibility of significant changes in chemical concentrations during the long term after a postulated loss-of-coolant accident (LOCA).

As stated on April 16, 1975, dose calculations had not been completed at that time to determine the feasibility of the operation of manual valves in the Auxiliary Building in the post-LOCA environment. Such operation would be necessary in order to implement the operating procedures described for Modes 1 and 2. These dose calculations have now been completed and indicate that, in the Auxiliary Building area where the maximum (controlling) exposure would be received, the minimum dose rate in the interval from initiation of recirculation to 60 days following a postulated LOCA is in excess of 40 R/hr. Other analyses have shown that if the leakage gaps between the outlet nozzles and the core support shield, as described in Supplement 1 to Topical Report BAW-10091, are assumed not to be available, that the previously mentioned operating modes would need to be implemented not later than 60 days following a postulated LOCA. It is apparent, therefore, that the manual operation of the various valves is not possible due to the extremely high exposures which would be involved.

In light of the above, a study was conducted to determine the feasibility of modifying the affected valves such that they could be remotely operated.

5965

This study has indicated that following initiation of procurement, a delivery time of approximately 21 months can be anticipated. When the additional time necessary for installation of the valve motor operators is included, the previous estimate of 2½ years for implementation of such a modification is confirmed.

In view of the above, and since the methods described in BAW-10091, Supplement 1, are adequate to prevent boron precipitation in the long term following a postulated LOCA, it is concluded that further consideration of implementation of the various described operating modes is neither practical nor necessary.

Very truly yours,

A. C. Thies

ACT:vr