

JUN 10 1980

MEETING SUMMARY DISTRIBUTION

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BCC: Applicant & Service List

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

JUN 10 1980

Docket Nos.: 50-369
and 50-370

APPLICANT: Duke Power Company
FACILITY: McGuire Nuclear Station, Unit Nos. 1 and 2
SUBJECT: SUMMARY OF MEETING HELD ON JUNE 6, 1980

A management meeting was held with the applicant on June 6, 1980 in Bethesda, Maryland, to discuss the status of the McGuire application. A list of attendees is shown in Enclosure No. 1.

The applicant stated that final hot functional testing of the Unit No. 1 would begin on June 7, 1980, and that initial fuel loading is scheduled for August 25, 1980. The applicant was informed that the staff review was proceeding on the basis of issuance of a full power operating license. Enclosure No. 2 is a listing of the remaining outstanding non-TMI items. The listing includes those items that require applicant action. It was mutually agreed that this list accurately reflected the remaining non-TMI items.

The NRC division and assistant directors briefly discussed the staff approach in its review of items stemming from TMI-2. The applicant agreed with the staff regarding the hydrogen generation aspect in an ice condenser plant. In this regard, the applicant stated that the intervenor may petition the ASLB to reopen the hearing. Their action would delay fuel loading well beyond August 1980.

B. J. Youngblood, Chief
Licensing Branch No. 1
Division of Licensing

Enclosures: As stated

Attendance List

McGuire Nuclear Station, Unit Nos. 1 and 2

June 6, 1980

Duke Power Company

G. A. Copp
K. S. Carady
W. O. Parker
M. D. McIntosh
C. J. Wylie
H. B. Tucker

NRC Staff

R. A. Birkel, DL
V. A. Moore, DHFS
P. S. Check, DSI
V. S. Noonan, DE
R. H. Vollmer, DE
R. A. Purple, DL
D. G. Eisenhut, DL
R. L. Tedesco, DL
B. J. Youngblood, DL
R. L. Ballard, DE
L. S. Rubenstein, CCS
D. B. Vassallo, DHFS
S. H. Hanauer, DHFS
D. F. Ross, DSI

June 5, 1980

OUTSTANDING NON-TMI ITEMSMcGUIRE REVIEW

<u>Generic Issues Branch</u>	1. ATWS
<u>Reactor Systems Branch</u>	1. SIS Bypass (UHI/ECCS bypass accident anal.) *2. LOCA Analysis; NRC 4/9/80 ltr. 3. BTP RSB 5-1 (RHR design req'ts)
<u>Equipment Qualification Branch</u>	*1. Environmental Qualification-Gen. (NUREG-0588)
<u>Mechanical Engineering Branch</u>	*1. Dynamic piping analysis (ISI, APP 3P, etc.) 2. Bolted Connections
<u>Containment Systems Branch</u>	1. CBS Tech Specs 2. Main Steam Line Break Anal. 3. Containment personnel airlock design; NRC 4/16/80 ltr.
<u>Effluent Treatment Systems Branch</u>	1. Radiation Effluent Tech Specs *2. Process control manual
<u>Materials Engineering Branch</u>	*1. Underclad cracking; NRC 5/12/80 ltr.
<u>Chemical Engineering Branch</u>	1. Secondary Water Chemistry
<u>Instrumentation & Control System Branch</u>	1. S. G. Level Instrument *2. Elect. Separation-field audit; NRC 4/9/80 ltr. 3. Environmental Qualifications-Barton #351 *4. Class IE equipment-skimmer, containment fan motor; NRC 3/7/80 ltr.
<u>Radiological Assessment Branch</u>	1. Offsite Dose Calculation Manual *2. Post-accident monitoring; NRC 6/4/80 ltr.
<u>Quality Assurance Branch</u>	1. Q-List
<u>Emergency Preparedness Licensing Branch</u>	1. Emergency Plan
<u>NMSS</u>	1. Security Plan *2. Security training & qualification plans; NRC 5/2/80 ltr.
<u>Licensing Guidance Branch</u>	1. Radiological Tech Specs
<u>Environmental Engineering Branch</u>	1. Non-radiological Tech Specs
<u>Auxiliary Systems Branch</u>	1. Standby Shutdown Facility evaluation *2. AFS reliability study; NRC 3/10/80 ltr.
<u>Power Systems Branch</u>	*1. Elect. system pre-op testing (voltage tap setting, etc.); NRC, 2/13/80 ltr.

*Denotes DPCo Action Item

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