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May 2, 1980

Mr. Karl Seyfrit
Director, Region IV
Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suit 1000
Arlington, Texas 76012

Subject: Georgia Power Company
Alvin W. Vogtle Nuclear Plant
Bechtel Job 9510-001
Potential Substantial Safety
Hazard/Significant Deficiency Report
File X7BG03 BQ 0004

Dear Mr. Seyfrit:

This report is an interim follow-up to our January 31, 1980 report to you under 10 CFR 21 requirements concerning a possible deficiency in pipe support design.

Our January 31, 1980 report anticipated completion of the entire corrective action program in approximately 3 months. Although major portions of the corrective action program have been completed, some pipe support designs remain to be reviewed. We now anticipate completion of review by June 1980, and filing of a final report to you by mid-July 1980. The following is the status of the Corrective Action program described under Section 7 of our January 31, 1980 report.

A. Actions by the Project

1. No pipe supports have been installed on Unit 1 or Unit 2. Work will be allowed to commence as pipe support designs are reviewed, revised as necessary, and issued for construction as described under paragraph 3 following.

2. New embed plate load capacity tables have been prepared and issued.

   The new Load Tables include:

   a. The effects of partial engagement and lateral loads.

b. Appropriate stiffening details at the embed plate and pipe support interface to assure full engagement where required.

c. Expanded scope to consider various loading conditions encountered in the design of the supports.

d. Revised criteria for attaching members parallel to the plane of the insert plate. The target area for attaching members perpendicular to the plane of insert plate has been extended to cover areas up to one inch from the edge of the embed plate.

3. The Phase I review of approximately 5,760 issued supports has been completed. Phase I review was to identify supports which require reanalysis and/or design modification, as a result of partial engagement or due to additional lateral loads, and supports that do not require further analysis or modification. 2,824 supports were found acceptable, requiring no analysis or modifications. These are being released for construction.

   Phase II review, including the reanalysis and any actions to carry out necessary design modifications utilizing the new Load Carrying Capacity Tables referenced above, is in progress. The current status of the detail review is as follows:

   a. Approximately 1,105 supports have been revised and are being reissued for construction. Of these supports:

      o 63% required reanalysis only, but no modification to the design;

      o 37% required reanalysis and redesign.

   b. The remaining 1,834 supports currently undergoing review will be issued after they have been reanalyzed and/or revised to meet the design requirements.

   c. This work is expected to be completed by June 1980.

4. Training sessions have been held between the Pipe Stress and Support Group and the Civil/Structural Design Group on proper use of the embed load capacity tables and correct interfacing of pipe supports to embed plates.

B. Actions by Divisional Management

1. The applicable design guide has been reviewed and found to be correct. No further action is required.

2. A Problem Alert was issued to all Bechtel projects requiring review of their pipe support design for correct interpretation and implementation of the design guide. Twenty-Six Bechtel nuclear projects, present and past, were alerted and responses received. All projects which had used the design guide reported correct interpretation and implementation. All projects which had not used the design guide reported that their design methods had properly handled the pipe support/embed interface using basic structural design methods. Two projects have not yet commenced pipe support design, but are now alerted to this incident. No further action is required.

We trust that this interim report will prove satisfactory. You may expect our final report in July.

Very truly yours,

BECHTEL POWER CORPORATION

B. L. Lex
Project Manager
Los Angeles Power Division

BLL/amt

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January 31, 1980

Mr. Karl Seyfrit
Director, Region IV
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 1000
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Alvin W. Vogtle Nuclear Plant
Bechtel Job 9510-001
Potential Substantial Safety
Hazard/Significant Deficiency Report
File X7BG03 Log BO 0003

Dear Mr. Seyfrit:

This is an interim report covering requirements under 10 CFR 21 and 10 CFR 50.55(e) regarding the potential deficiency reported to you by telephone at approximately 8:50 A.M. Pacific Standard Time on January 29, 1980 by Bechtel personnel, Burton L. Lex, Project Manager Vogtle. This same condition was reported on January 28, 1980 to Inspection and Enforcement, Region II (Atlanta) by telephone by Georgia Power Company as a significant reportable deficiency per 10 CFR 50.55(e) as reported to you on January 29, 1980. Bechtel has concluded that this condition may be reportable under 10 CFR 21.

This report is intended to describe the condition, our investigation and conclusions reached to-date, and our corrective action program to complete the evaluation and to resolve any defects in design. Descriptive material required under both
are included.

The following is the specific in

1. INDIVIDUAL REPORTING

Burton L. Lex, Project Manag
Bechtel Power Corporation,
90650

DUPLICATE DOCUMENT

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