NRC FORM 366 (7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

8005080458

CONTROL BLOCK: ① (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Value
01	Licensee Code ②	NYJAF1
01	License Number ③	00-0000-000
01	License Type ④	41111
01	CAT ⑤	
01 (CON'T)	Report Source ⑥	L
01	Docket Number ⑦	05000333
01	Event Date ⑧	040580
01	Report Date ⑨	050180

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

02 Drywell to torus differential pressure did not meet the 1.7 psi requirement of

03 T.S. 3.7.A.7 for approximately 35 minutes. T.S. allows six (6) hours to restore

04 differential pressure. No significant hazard existed. See attachment for

05 details.

Line 09	Value
System Code ⑪	SA
Cause Code ⑫	X
Cause Subcode ⑬	Z
Component Code ⑭	ZZZZZZ
Comp. Subcode ⑮	Z
Valve Subcode ⑯	Z

LER/RO REPORT NUMBER ⑰

Event Year		Sequential Report No.		Occurrence Code	Report Type		Revision No.
80	—	031	/	03	L	—	0

Action Taken ⑱	Future Action ⑲	Effect on Plant ⑳	Shutdown Method ㉑	Hours ㉒	Attachment Submitted ㉓	NPRD-4 Form Sub. ㉔	Prime Comp. Supplier ㉕	Component Manufacturer ㉖
X	Z	Z	Z	0000	Y	N	Z	Z999

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ㉗

10 A rapid temperature decrease in the system which cools the drywell resulted in

11 the differential pressure of 1.66 psid compared to a limit of equal to or greater

12 than 1.70. See attachment for details.

Line	Field	Value
15	Facility Status ㉘	E
15	% Power ㉙	079
15	Other Status ㉚	NA
15	Method of Discovery ㉛	A
15	Discovery Description ㉜	Operator Observation
16	Activity Released ㉝	Z
16	Content of Release ㉞	Z
16	Amount of Activity ㉟	NA
16	Location of Release ㊱	NA
17	Personnel Exposures Number ㊲	000
17	Type ㊳	Z
17	Description ㊴	NA
18	Personnel Injuries Number ㊵	000
18	Description ㊶	NA
19	Loss of or Damage to Facility Type ㊷	Z
19	Description ㊸	NA
20	Publicity Issued ㊹	N
20	Description ㊺	NA

NRC USE ONLY

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POWER AUTHORITY OF THE STATE OF NEW YORK
JAMES A. FITZPATRICK NUCLEAR POWER PLANT

DOCKET NO.50-333

ATTACHMENT TO LER 80-031/03L-0

During normal operation, the Drywell to Torus Differential Pressure was less than the requirements of Technical Specifications paragraph 3.7.A.7. This resulted in operation in a degraded mode allowed by paragraph 3.7.A.7.A.3 for a period of approximately 35 minutes.

The differential pressure was out of specification approximately 0.04 psid due to a rapid drop in the reactor building cooling system temperature. An operator initiated trip of the reactor water cleanup system (due to a valve packing failure) resulted in a sudden decrease in the heat load on the reactor building cooling system which in turn resulted in a rapid decrease in the temperature of the reactor building cooling system. This temperature decrease resulted in a drywell pressure decrease (due to cooling). Immediate restoration of the Reactor Building Cooling Water temperature to its normal range restored the drywell to torus differential pressure to within technical specification limits. To provide the operator with early warning that drywell to torus differential pressure is approaching the limit, the low differential pressure alarm has been raised to 1.75 psid. In addition, the reactor building cooling system temperature alarm setpoint will be changed during the 1980 refueling outage to provide the operator with early warning of decreasing temperature.

No further action will be required to reduce the probability of recurrence. The event did not represent a significant hazard to the public health and safety.

NOTE: LER 80-017 is a related event.