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April 29, 1980

Mr. A. Bournia
Division of Project Management
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: LaSalle County Station Units 1 and 2
Design Assessment Report (Rev. 8 - Draft)

Dear Mr. Bournia:

As discussed in our telephone conversation of April 28, 1980, I am forwarding certain pages from the subject report that were inadvertently omitted when it was submitted by Reference (a). These pages will be formally incorporated into the next amendment to the DAR.

If you should have any questions, please contact me.

Sincerely,

L. O. DelGeorge

L. O. DelGeorge
Nuclear Licensing Administrator

Attachment

cc: J. S. Abel (CECo.)

B001
S, 1/1

LOAD OR PHENOMENON	MARK II OWNERS GROUP LOAD SPECIFICATION	NRC REVIEW STATUS	LA SALLE POSITION ON ACCEPTANCE CRITERIA
b) Medium Steam Flux Loads	Sinusoidal pressure fluctuation added to local hydrostatic. Amplitude uniform below vent exit-linear attenuation to pool surface. 7.5 psi peak-to-peak amplitude. 5, 6 Hz frequencies.	Acceptable	_____
c) Chugging Loads	Representative pressure fluctuation taken from 4T test added to local hydrostatic.	Acceptable pending resolution of FSI concerns.	_____
- uniform loading condition	Maximum amplitude uniform below vent exit-linear attenuation to pool surface. +4.8 psi maximum overpressure, -4.0 psi maximum under pressure, 20-30 Hz frequency.		
- asymmetric loading condition	Maximum amplitude uniform below vent exit-linear attenuation to pool surface. 20 psi maximum overpressure, -14 psi maximum underpressure, 20-30 Hz frequency, peripheral variation of amplitude follows observed statistical distribution with maximum and minimum diametrically opposed.		

3.4-4c

POOR ORIGINAL

LOAD OR PHENOMENON	MARK II OWNERS GROUP LOAD SPECIFICATION	NRC REVIEW STATUS	LA SALLE POSITION ON ACCEPTANCE CRITERIA
IV. Secondary Loads			
A. Sonic Wave Load	Negligible Load - none specified	Acceptable	_____
B. Compressive Wave Load	Negligible Load - none specified	Acceptable	_____
C. Post Swell Wave Load	No generic load provided	Plant unique load specification and NRC review.	Described in La Salle Closure Report
D. Seismic Slosh Load	No generic load provided	Plant unique load specification and NRC review.	Described in La Salle Closure Report
E. Fallback load on Submerged Boundary	Negligible load - none specified	Acceptable	_____
F. Thrust Loads	Momentum balance	Acceptable	_____
G. Friction Drag Loads on Vents	Standard friction drag calculations	Acceptable	_____
H. Vent Clearing Loads	Negligible Load - none specified	Acceptable	_____

3.4-9

POOR ORIGINAL

LOAD OR PHENOMENON	MARK II OWNERS GROUP LOAD SPECIFICATION	NRC REVIEW STATUS	LA SALLE POSITION ON ACCEPTANCE CRITERIA
FUNCTIONAL CAPABILITY		Interim technical position (7/19/78)	Acceptable. Rodabaugh criteria may be used in some cases if NRC finds acceptable.
MASS-ENERGY RELEASE FOR ANNULUS PRESS.		Verify using RELAP*/ MAD*	Acceptable
QUESTIONS MEB-2, MEB-5		ISI peak broadening to be used.	Acceptable
MEB-3, MEB-5		Closely spaced modes combined Per 1.92	Acceptable. NSSS scope uses modified summation per approved GESSAR.
MEB-1		Dynamic analysis methods acceptable	Acceptable
MEB-2		OBE Damping - Level A or B SSE Damping - Level C or D	Acceptable
MEB-6		Seismic slosh-plant unique review	Acceptable
MEB-7a and b		Load Combinations: AP+SSE OBE+SRV	Acceptable. See load combination table for Case #2 and 7
MEB-8		Functional capability and piping acceptance criteria	See load combination table.

POOR ORIGINAL

LOAD PHENOMENON	MARK II OWNERS GROUP LOAD SPECIFICATION	NRC REVIEW STATUS	LA SALLE POSITION ON ACCEPTANCE CRITERIA
1.		$N+SRV_x$ To B	Acceptable
2.		$N+SRV_x$+OBE to B	Acceptable Approved GESSAR approach used for NSSS.
3.		$N+SRV_{all}$+SSE to C	Acceptable
4.		$N+SRV_{ads}$+OBE+IBA to C	Acceptable
5.		$N+SRV_{ads}$+OBE+IBA to C	Acceptable
6.		$N+SRV_{ads}$+SSE+IBA to C	Acceptable
7.		N+SSE+DBA to C	Acceptable
8.		N to A	Acceptable
9.		N+OBE to B	Acceptable
10.		$N+SRV_a$+SSE+DBA to C	Applied to containment structure only (See M 020.22 and DFTR 5.2.4, and letter to R. J. Mattson from L. J. Sobon dated Feb. 22, 1979).

3.4-11
POOR ORIGINAL

Include this page!!

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Event 3 is a stuck-open relief valve with the reactor isolated from the main condenser and maintained at operating pressure and temperature. All RHR heat exchanger equipment is considered operational in this case because if one heat exchanger is not available, the reactor may not remain isolated and pressurized. The bulk pool temperature assumed in this event is the highest allowed while the reactor is maintaining pressure.

Event 4 is a controlled depressurization from the plant condition described in Event 3. This event demonstrates the ability of the plant to safely depressurize through a controlled transient from the limiting conditions.

Event 5 is a controlled depressurization several minutes after the reactor isolation described in Event 3. One heat exchanger is considered unavailable because power operation in that condition is not prohibited. This event demonstrates the ability to depressurize the plant safely through a controlled transient with only partial RHR heat exchanger equipment availability.

Event 6 is a rapid depressurization of the reactor resulting from actuation of the automatic depressurization system (ADS). ADS actuation takes place at the time the reactor is scrammed. Because this event takes place quickly, no heat exchangers are brought into operation. This event represents the most rapid energy release to the pool and demonstrates that the plant may be safely depressurized in this manner without use of heat exchangers.

Event 7 is also a rapid depressurization via ADS but with the reactor isolated and pressurized and with a limiting pool temperature condition. All heat exchanger equipment is placed in the pool cooling mode 1/2 hour after scram but is considered unavailable when depressurization begins.