

NSP

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

RR #2

Welch, MN 55089

April 30, 1980

Mr. James G Keppler, Director
Region III Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, IL 60137

ATTN: D C Boyd

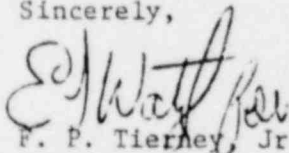
Reportable Occurrence No. 80-15

On April 29, 1980, a volumetric test of the Unit 1 containment maintenance airlock showed leakage of about 600 scc/min. The Tech Spec limit is 409 sccm. The seals for the inner door were replaced and leakage was acceptable.

The volumetric test pressurizes the space between the doors to 46 psig. Thus, the test pressure tends to open the inner airlock door. Also, door stops are installed prior to the test to prevent overcompression of (and damage to) the door seals. For these reasons it is concluded that it is quite possible that door leakage would have been acceptable during the postulated accident. However, this was not proved.

The outer door was operable throughout the period.

Sincerely,



F. P. Tierney, Jr.,
Plant Manager
Prairie Island Nuclear Plant

FPT:dlv

cc: L O Mayer
Director, MIPC (2)
% Distribution Services Branch, DDC, ADM
C D Feierabend

Acc 2
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