

CONTROL BLOCK: _____ (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | M | D | C | C | N | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T
0 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 1 | 8 | 7 | 0 | 1 | 0 | 6 | 7 | 9 | 8 | 0 | 4 | 2 | 1 | 8 | 0 | 9
7 8 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | At 0900 Reactor Coolant System (RCS) unidentified leakage was 1.546 gpm, exceeding _____
0 3 | the amount allowed by T.S. 3.4.6.2.C. At 0930 commenced power decrease to _____
0 4 | investigate leakage around 21A reactor coolant pump (RCP). 21A RCP middle _____
0 5 | seal pressure sensing line was found to have a cracked weld and was isolated. _____
0 6 | RCS leakage was determined to be less than 1 gpm at 1448. LER's 77-15 (U-1) _____
0 7 | and 78-50 (U-2) describe similar events. _____

0 8 | _____ 80

0 9 | C | B | 11 | B | 12 | A | 13 | P | I | P | E | X | X | 14 | A | 15 | Z | 16
9 10 11 12 13 14 15 16 17 18 19 20
17 | LER RO | EVENT YEAR | SEQUENTIAL | OCCURRENCE | REPORT | REVISION
REPORT | NUMBER | 7 | 9 | REPORT NO. | CODE | TYPE | NO.
NUMBER | 21 | 22 | 0 | 0 | 1 | 0 | 3 | Y | 1
23 | 24 | 26 | 27 | 28 | 29 | 30 | 31 | 32
18 | F | 18 | Z | 19 | A | 20 | A | 21 | 0 | 0 | 2 | 4 | Y | 23 | N | 24 | A | 25 | B | 5 | 8 | 0 | 26
33 34 35 36 37 40 41 42 43 44 47
22 | HOURS | 22 | 0 | 0 | 2 | 4 | Y | 23 | N | 24 | A | 25 | B | 5 | 8 | 0 | 26
37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Upon investigation the cracked weld was found to be located on a socket weld _____
1 1 | downstream of the second isolation valve. Weld repairs were performed for _____
1 2 | return to service. Modified sensing lines and supports have been installed _____
1 3 | and further problems in this area are not expected. _____

1 4 | _____ 80

1 5 | FACILITY | % POWER | OTHER STATUS | 30 | METHOD OF | DISCOVERY DESCRIPTION | 32
STATUS | 0 | 9 | 7 | 29 | NA | B | 31 | Operator Observation
7 8 9 10 12 13 44 45 46 80

1 6 | ACTIVITY | CONTENT | AMOUNT OF ACTIVITY | 35 | LOCATION OF RELEASE | 36
RELEASED OF RELEASE | Z | 33 | Z | 34 | NA | NA
7 8 9 10 11 44 45 80

1 7 | PERSONNEL EXPOSURES | DESCRIPTION | 39
NUMBER | TYPE | 0 | 0 | 0 | 37 | Z | 38 | NA
7 8 9 10 11 12 13 80

1 8 | PERSONNEL INJURIES | DESCRIPTION | 41
NUMBER | 0 | 0 | 0 | 40 | NA
7 8 9 10 11 12 80

1 9 | LOSS OF OR DAMAGE TO FACILITY | DESCRIPTION | 43
TYPE | Z | 42 | NA
7 8 9 10 80

2 0 | PUBLICITY | DESCRIPTION | 45
ISSUED | N | 44 | NA
7 8 9 10 80

NRC USE ONLY

FOLLOW-UP
LER NO. 79-01/3L
DOCKET NO. 50-318
EVENT DATE 01/06/79
REPORT DATE 04/25/80
ATTACHMENT

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS:

Upon investigation the cracked weld was found to be located on a socket weld downstream of the second isolation valve. Weld repairs were performed for return to service. A vibration analysis performed during the past re-fueling outage determined that fatigue levels of vibration existed at several locations on RCP instrument piping/tubing. Braces added to this piping reduced but did not eliminate vibration. An engineering study was carried out to determine system change requests (FCR's: 77-147 & 78-1028) have been completed. Observation of these lines will continue, but the problem appears to have been eliminated.