

**Florida  
Power**  
CORPORATION

April 20, 1980

File: 3-0-3-a-3

Mr. Robert W. Reid  
Chief  
Operating Reactors Branch #4  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Subject: Crystal River Unit 3  
Docket No. 50-302  
Operating License No. DPR-72

Dear Mr. Reid:

On February 15, 1980, Florida Power Corporation submitted a response concerning its implementation of NUREG-0578, Short-term Lessons Learned Recommendations, and the Commission's January 2, 1980 Show Cause Order.

Since that time we have had several discussions with Mr. Chuck Long of the NRC staff concerning our response. At the request of Mr. Long, Florida Power Corporation hereby submits the attached additional information regarding our implementation of NUREG-0578.

If you have any further questions regarding this submittal, please contact this office.

Very truly yours,

FLORIDA POWER CORPORATION

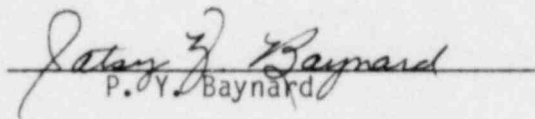
P. Y. Baynard  
Manager  
Nuclear Support Services

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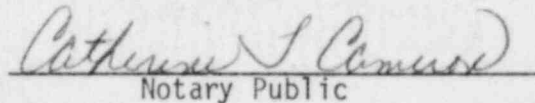
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STATE OF FLORIDA  
COUNTY OF PINELLAS

P. Y. Baynard states that she is the Manager, Nuclear Support Services of Florida Power Corporation; that she is authorized on the part of said company to sign and file with the Nuclear Regulatory Commission the information attached hereto; and that all such statements made and matters set forth therein are true and correct to the best of her knowledge, information and belief.

  
P. Y. Baynard

Subscribed and sworn to before me, a Notary Public in and for the State and County above named, this 20th day of April, 1980.

  
Notary Public

Notary Public, State of Florida at Large,  
My Commission Expires: August 8, 1983

CameronNotary 3(D12)

ADDITIONAL INFORMATION REQUESTED BY THE NRC REGARDING NUREG-0578

Item 2.1.3.a - Direct Indication of Power-operated Relief Valve and Safety Valves for PWRs:

Florida Power Corporation is participating with Babcock & Wilcox in a generic program to environmentally qualify the valve monitoring equipment. This program is presently scheduled to be completed by October, 1980. Should there be any change in the schedule of this program, we will notify you immediately.

Item 2.1.3.b - Subcooling Meter:

The  $T_{sat}$  meter as presently designed has two hot leg and two cold leg temperature inputs. The  $T_{sat}$  meter is powered directly from a vital bus and not through the NNI system.

In addition to the above temperature inputs, FPC is also providing the hottest of 5 core exit T/Cs to each  $T_{sat}$  meter. These T/Cs are selected from each core quadrant and the central region. This addition will give the operator the capability to selectively observe saturation margin of the hottest T/C,  $T_h$  or  $T_c$  against pressure in either loop. The alarm on reduced saturation margin will be from the hottest temperature input and selected loop pressure. Installation of the modification is scheduled to be completed by June 1, 1980.

Item 2.1.5.a - Dedicated Penetration for External Recombiners or Post-Accident Purge Systems:

Crystal River Unit 3 has a redundant method of pressurizing the Reactor Building (RB) for Post-Accident Hydrogen Purging (FD-302-722) by using the Station Air System with valve SAV-51 in the open position (see FD-302-271, FSAR Figure 9-16). This redundant system (SA) was successfully used to purge the RB following our February 26, 1980 incident. This additional system has a flow rate of 100-150 cfm which is consistent with our purging rates. The Station Air System and the Hydrogen Purge System provide a redundant means for Post-Accident purging and satisfy the single-failure criteria of NUREG-0578.

Item 2.1.6.a - Integrity of Systems Outside Containment Likely to Contain Radioactive Material for PWRs and BWRs:

Leak rate testing of the Decay Heat System, Reactor Building Spray System and the Liquid Sampling System will be completed during our present refueling outage. The leak rate values measured for these systems will be submitted to the NRC per NUREG-0578.

Item 2.1.6.b - Design Review of Plant Shielding and Environmental Qualification of Equipment for Spaces/Systems Which May Be Used in Post-Accident Operations.

In our January 11, 1980 submittal, we identified certain modifications necessary to resolve post-accident access problems to certain plant areas. Also included in our response were tables identifying equipment locations, radiation-sensitive materials within these locations, and the integrated dose calculated for the location.

Prior to June 1, 1980, FPC will identify any unacceptable component/or material for mechanical equipment within these areas and provide our proposed corrective action to solve the problem. For material in electrical components, this evaluation is being accomplished as part of our review of I.E. Bulletin 79-01B. As stated in our April 17, 1980 letter to Mr. J. P. O'Reilly, this effort will be completed by June 30, 1980. Following completion of this effort, we will provide a list of the unacceptable components/or material for electrical equipment to you along with our proposed corrective action.

Item 2.1.8.b - Increased Range of Radiation Monitors.

In addition to the procedures identified in our February 15, 1980 submittal for quantifying plant releases from potential release points, steam line monitors are required per the NRC. These monitors will be provided for each steam generator on the steam line upstream of the mainsteam safety and atmospheric dump valves. The monitors will be located outside containment and will have a local readout. The detectors will be provided with circuitry which provides local first-out alarm indication of a significant tube leak to identify the affected steam generator.

The development of this system has been undertaken by FPC, with assistance from our consultant, Applied Physics Technology (APT). An installation target date of June 1, 1980 has been identified and delivery commitment dates have been requested from suppliers. Although we have not received any firm commitment dates, no items have been identified as implementation restrictions. If any item becomes a restraint to our June 1, 1980 installation deadline, we will notify you immediately and provide compensatory measures until this modification can be completed.

Item 2.1.8.c - Improved In-Plant Iodine Instrumentation Under Accident Conditions.

A portable G-M detection and counting system as identified in our February 15, 1980 submittal will be located in the Control Room and Technical Support Center.

We are also planning the use of either a portable Cadmium Telluride or Sodium Iodide Detection System, coupled to a single-channel analyzer. One of these portable systems will be located in both the Control Room and the Technical Support Center.