

LICENSEE EVENT REPOF

CONTROL BLOCK: [] [] [] [] [] [] [] [] [] []

(PLEASE PRINT ALL REQUIRED INFORMATION)

LICENSEE NAME: AIRARKI LICENSE NUMBER: 00-000000-00 LICENSE TYPE: 411111 EVENT TYPE: 03

CATEGORY: CONT REPORT TYPE: L REPORT SOURCE: L DOCKET NUMBER: 050-0313 EVENT DATE: 072677 REPORT DATE: 082577

EVENT DESCRIPTION

02 AT STEADY-STATE POWER (100% FP) A SURVEILLANCE TEST INDICATED THREE PENETRATIONS WITH A HIGH LEAKAGE RATE. IT WAS DETERMINED THAT THE MAXIMUM LEAKAGE RATE WAS GREATER THAN 60% L_a. THIS IS NOT A REPETITIVE OCCURRENCE. THE LEAKAGE WAS REDUCED BY REPAIRING THE ISOLATION VALUES. (50-313/77-15)

SYSTEM CODE: SD CAUSE CODE: E COMPONENT CODE: PENETR PRIME COMPONENT SUPPLIER: A COMPONENT MANUFACTURER: Zc a a z VIOLATION: N

CAUSE DESCRIPTION

08 A SURVEILLANCE TEST INDICATED A HIGH LEAKAGE RATE ON THREE PENETRATIONS. IT APPEARS THAT FOREIGN MATTER MAY HAVE CAUSED THE ISOLATION VALUES NOT TO PERFORM PROPERLY. THE LEAKAGE WAS REDUCED BY REPAIRING THE ISOLATION VALUES.

11 FACILITY STATUS: E % POWER: 100 OTHER STATUS: NA METHOD OF DISCOVERY: b DISCOVERY DESCRIPTION: NA

12 FORM OF ACTIVITY RELEASED: Z CONTENT OF RELEASE: Z AMOUNT OF ACTIVITY: NA LOCATION OF RELEASE: NA

PERSONNEL EXPOSURES

13 NUMBER: 000 TYPE: Z DESCRIPTION: NA

PERSONNEL INJURIES

14 NUMBER: 000 DESCRIPTION: NA

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

16 TYPE: Z DESCRIPTION: NA

PUBLICITY

17 NA 8004110 721

ADDITIONAL FACTORS

18 IN THE SOLUTION OF THE NRC'S UNRESOLVED ITEM 77-03, IT WAS DETERMINED THAT THE MAXIMUM TESTAGE LEAKAGE WAS GREATER THAN 60% L_a.

NAME: DAVID G. MEDIS PHONE: 501/371-4496

1. Reportable Occurrence Report No. 50-313/77-15
2. Report Date: 8/25/77 3. Occurrence Date: 7/26/77
4. Facility: Arkansas Nuclear One - Unit 1
 Russellville, Arkansas 72801

5. Identification of Occurrence:

Exceeding the allowable leakage from penetrations per
 Tech. Spec. 4.4.1.2.3.

6. Conditions Prior to Occurrence:

Steady-State Power	<u> X </u>	Reactor Power	<u> 2568 </u>	MWth
Hot Standby	<u> </u>	Net Output	<u> 773 </u>	MWe
Cold Shutdown	<u> </u>	Percent of Full Power	<u> 100 </u>	%
Refueling Shutdown	<u> </u>	Load Changes During Routine Power Operation	<u> </u>	
Routine Startup Operation	<u> </u>			
Routine Shutdown Operation	<u> </u>			
Other (specify)				

7. Description of Occurrence:

During leak rate testing of reactor building penetrations in February 1977, the Instrument Group found and repaired three penetrations with leakage great enough that the leakage rate could not be determined. The Instrument Group was working under the assumption that the maximum leakage that could be tested was less than 60% La. In the solution of the NRC's unresolved item 77-03, (I&E Inspection Report No. 50-313/77-03) it was determined that the maximum testable leakage was greater than 60% La and that we may have been in violation of Tech. Spec. 4.4.1.2.3.

8. Designation of Apparent Cause of Occurrence:

Design	_____	Procedure	_____
Manufacture	_____	Unusual Service Condition Including Environmental	_____
Installation/ Construction	_____	Component Failure (See Failure Data)	_____
Operator	_____		
Other (specify)	X		

It appears that foreign matter may have caused the isolation valve to not perform properly.

9. Analysis of Occurrence:

Since the surveillance testing indicated that the penetration isolation valves were leaking and the leakage was reduced to allowable limits prior to establishing containment integrity, there was no hazard to the health and safety of the public.

Reportable Occurrence Report No. 50-313/77-15

10. Corrective Action:

The leakage was reduced to an allowable limit by repairing the isolation valves for the penetrations in questions (No. S1, V-1 and V-2).

11. Failure Data:

None.