

U.S. NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT

Region I

Report No. 50-219/79-24

Docket No. 50-219

License No. DPR-16 Priority -- Category C

Licensee: Jersey Central Power and Light Company

Madison Avenue at Punch Bowl Road

Morristown, New Jersey 07960

Facility Name: Oyster Creek, Unit 1

Inspection at: Forked River, New Jersey

Inspection conducted: December 11-14 and 18, 1979

Inspectors: L. Briggs  
L. Briggs, Reactor Inspector

1/25/80  
date signed

P. Koltay  
P. Koltay, Reactor Inspector

1/25/80  
date signed

Approved by: R. R. Keimig  
R. R. Keimig, Chief, Reactor Projects Section  
No. 1, RO&NS Branch

1-25-80  
date signed

Inspection Summary:

Inspection on December 11-14 and 18, 1979 (Report No. 50-219/79-24)

Areas Inspected: Routine, unannounced inspection by two region-based inspectors (55 hours) of 5 inspection areas: followup on previous inspection findings; independent inspection effort; IE Bulletin followup; review of periodic reports; and, fire protection.

Noncompliances: One: Storage of flammable material on the 119 foot elevation of the reactor building (Paragraph 4).

## DETAILS

### 1. Persons Contacted

- B. Bollinger, Project Superintendent, Electrical, Riggs-Distler
- #\*J. Carroll, Station Superintendent
- K. Eichenlaub, Fire Protection Specialist
- \*W. Garvey, Director, Station Administration
- J. Lazar, Engineering, GCMD
- \*C. Lefler, Radioactive Waste Superintendent
- T. McCluskey, Supervisor, GCMD
- #D. Mencinelli, Project Engineer
- \*\*D. Ross, Manager, Generating Stations, Nuclear
- T. Spence, Project Engineer, GCMD
- #\*J. Sullivan, Unit Superintendent
- \*D. Turner, Supervisor, Health Physics

The inspector also contacted other members of the technical, engineering, and operating staff.

- \*Present at the exit interview on December 14, 1979.
- \*\*Present at the exit interview on December 14, 1979, via telephone.
- #Present at the exit interview on December 18, 1979.

### 2. Licensee Action on Previous Inspection Findings

(Closed) Follow Item (219/79-11-24): Operator Training on Procedure Change as a Result of IE Bulletin 79-08 and the May 2, 1979, Lo-Lo-Lo Water Level Event. The inspector verified that procedures listed in the lesson plan encompassed all procedures identified in IE Inspection Report No. 50-219/79-11. Training sessions were conducted on May 24 and 31, and June 6, 14, and 22, 1979. All operations personnel, with 2 exceptions, attended one or more of the training sessions. The inspector verified that the 2 absent personnel had read the procedure revisions and signed the Required Reading Log. All procedure revisions are promulgated through the Required Reading Notebook on a continuing basis to ensure that operations personnel are cognizant of any changes.

No items of noncompliance were identified.

(Closed) Unresolved (219/79-11-08): Repair Locking Devices on Two Electrical Breakers. The inspector verified by physical inspection during a plant tour that the breaker locking devices (for valves V-20-27 and V-20-18) were repaired and fully functional.

No items of noncompliance were identified.

(Closed) Unresolved (219/79-11-01): GE Print No. 729E182, ADS Automatic Actuation Logic Not Revised to Show "C" Battery Distribution. The above schematic was revised September 15, 1979, and verified, by a review of microfilm prints in the control room, to contain correct references and identification of "C" Battery Distribution.

No items of noncompliance were identified.

(Closed) Unresolved (219/79-10-04): Resolution of 36 Minute Time Frame to Start a Feedwater Pump During the May 2 Event. Inability to start "A" feedwater pump was caused by a trip of the thermal overload relay. The licensee has permanently bypassed the pump breaker thermal overloads. Magnetic overloads are still operable providing overcurrent protection. Further studies are being performed under Engineering Task No. 294, Oyster Creek Feedwater System Reliability Study. Study completion date is scheduled for December 31, 1979.

No items of noncompliance were identified.

(Closed) Unresolved (219/79-06-01): Formal Method of Standing Order Implementation. Procedure No. 115, Standing Order Control, Revision 7, September 15, 1979, addresses Standing Order issuance and control. All Standing Orders are reviewed by PORC prior to issuance and quarterly, thereafter. Standing Orders are distributed as required reading.

No items of noncompliance were identified.

(Closed) Unresolved (219/78-24-03): Revise Procedure 320.1 to Show Demineralized Water (DI) Valve V-12-60 as Closed. Procedure No. 320.1, Demineralized Water Transfer System, Revision 3, September 3, 1978, requires V-12-60 to be closed. This little used section of the system is scheduled to have blank flanges installed on both sides of the drywell, under Modification Package No. 259-78-23, during the 1980 refueling outage.

No items of noncompliance were identified.

(Closed) Unresolved (219/78-16-02): Control of Temporary Procedure Changes. Procedure No. 107, Procedure Control Procedure, Revision 12, September 27, 1979, addresses control of permanent and temporary procedure changes. Form 103-1, Revision 3, Station Controlled Distribution Document Request, requires specific personnel (by job position and qualification) to approve procedure changes and denotes if the change is for a new document, temporary change, one time change or a procedure revision.

No items of noncompliance were identified.

(Open) Unresolved (219/79-11-19): Diesel Generator Lineup Procedure Revision. Procedure 341, Standby Diesel Generators, has been revised by the licensee, but had not been reviewed by PORC at the conclusion of this inspection. Procedure review is scheduled for completion by December 31, 1979.

This item remains open pending PORC review and subsequent NRC:RI review.

(Open) Follow Item (219/79-12-01): "A" and "B" Battery Breaker Position Not Annunciated in the Control Room. The licensee (site) initiated Engineering Request No. M79-029, June 14, 1979. Engineering Task No. 377.00 was assigned by Generation Engineering Department on June 21, 1979 and is currently in an active status.

This item will be reinspected by NRC:RI, subsequent to the completion of licensee action.

(Open) Unresolved (219/79-11-04): Licensee to Correct Fill and Vent Section of Procedure No. 304, Standby Liquid Control System. This procedure has been revised by the licensee but had not been reviewed by PORC prior to the conclusion of this inspection. Procedure review is scheduled for completion by December 31, 1979.

This item remains open pending PORC review and subsequent NRC:RI review.

3. Modification Required by Amendment No. 29 to Provisional Operating License No. DPR-16

By observation and examination of records, the inspector verified, on a sampling basis, that required modifications are being implemented as identified and scheduled in Table 3.1 of the Fire Protection Safety Evaluation Report (SER), issued as Amendment 29 to DPR-16, Paragraph 3.1.1 through 3.1.23. The licensee stated that all modifications will be completed by the end of the January, 1980 refueling outage, except items delayed in accordance with the licensee's request in a memorandum dated October 12, 1979.

An NRC:RI inspector will review the completed modifications during a subsequent inspection. This item is considered unresolved (79-24-01).

4. Facility Tour

The inspectors examined existing fire protection equipment including: fire alarm and fire suppression systems and fire barrier penetration seals in the control room complex, the cable spreading room, the 480V switchgear room, the 4160V switchgear area, battery rooms, throughout the reactor

building, transformer yard and the fire pump house. During the tour of the facility, the inspectors observed the storage of eleven wooden crates, containing CRD blades, on the refueling floor (119' elevation) of the reactor building. The inspectors noted that the crates were constructed from wood that had not been treated with flame retarding material.

Procedure No. 120, "Fire Hazards," Revision 5, dated October 12, 1979, Section 5.1.12, states: "All wood used in safety related areas will be fire retardant."

The inspector notified the licensee that the storage of untreated wooden crates on elevation 119' of the reactor building was contrary to Station Procedure No. 120. The licensee acknowledged the problem and stated that prompt corrective action would be taken. On December 18, 1979, the inspector reinspected the wooden crates and verified that the licensee had coated the outside surface of each crate with a fire retardant paint. This is an acceptable method of corrective action.

This is an item of noncompliance (219/79-24-02).

#### 5. In-Office Review

In-office review of the following reports has been completed with no unacceptable conditions identified:

- October, 1979 Monthly Statistical Report.
- November, 1979 Monthly Statistical Report.

#### 6. IE Bulletins

Licensee actions concerning the following IE Bulletins were reviewed by the inspector to verify that: the Bulletin was forwarded to appropriate onsite management; a review for applicability was performed; information discussed in the licensee's reply was accurate; corrective action taken was as described in the reply; and, the reply was within the time period described in the Bulletin.

- IEB 78-05, Malfunctioning of Circuit Breaker Auxiliary Contact Mechanism, General Electric Model CR105X. Review of Job Order No 0585E verified that auxiliary contact mechanisms in safety related 480V circuit breakers had been replaced as of January 23, 1979.

Quality controlled parts were used and documented under Procurement Quality Assurance (PQA) No. 78-596 with breaker testing witnessed by the Quality Assurance Department.

No items of noncompliance were identified.



- IEB 79-04, Incorrect Weights for Swing Check Valves Manufactured by Velan Engineering Corporation. The licensee's response was verified by a review of the Master Valve List, Burns and Roe Drawing No. 2075, reviewed March 13, 1979. The following systems were reviewed:
- Emergency Condenser;
  - Control Rod Drive Hydraulic;
  - Reactor Shutdown Cooling;
  - Liquid Poison;
  - Core Spray;
  - Containment Spray; and,
  - Fuel Pool Cooling.

No Velan swing check valves of the size concerned with the IEB were identified other than noted in the licensee's response.

No items of noncompliance were identified.

7. Unresolved Items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable, items of noncompliance, or deviations. The unresolved item identified during this inspection is discussed in Paragraph 3.

8. Exit Interview

The inspectors met with licensee representatives (denoted in Paragraph 1) at the conclusion of the inspection on December 14 and 18, 1979. The inspectors summarized the scope and findings of the inspection as described in this report.