



THE ELECTRIC COMPANY

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KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT-OPERATIONS

March 5, 1980

Mr. W.C. Seidle, Chief
Reactor Construction and
Engineering Support Branch
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

Re: STN 50-482
Subj: Interim Status Report - NSSS Lateral
Support Anchor Nuts

Dear Mr. Seidle:

The purpose of this letter is to supplement our previous notification of December 27, 1979, to you concerning difficulties experienced at the Wolf Creek Site with out-of-tolerance threads for the NSSS lateral support anchor bolt nuts. In that notification you were advised that additional inspections and analyses of selected anchor bolt nuts would be initiated in order to fully determine the extent and scope of these difficulties and, further, to assess the safety significance of this matter pursuant to 10CFR50.55(e) regulations. A follow up report by March 1, 1980 was indicated at that time.

Follow-up studies have since been initiated involving representative samples of nonconforming nuts identified at the two SNUPPS jobsites. To date over 900 - 3-1/2 inch diameter nuts have been inspected and pertinent out-of-tolerance conditions identified for further analyses. Three such nuts, representing worst case conditions identified between the two jobsites, have been tested at Lehigh University in accordance with test procedures approved by Bechtel Power Corporation. The results of these tests are currently being analyzed and will be compared with maximum bolt loads being calculated for a given support structure. Based upon these analyses, efforts will also be made to correlate analytical and empirical test data in order to more precisely establish methods for determining the strength of the threads on the nut. This effort is expected to be complete on or about June 1, 1980. Until this effort is complete, we are unable to conclude whether the described difficulties represent a Significant Deficiency as defined in 10CFR50.55(e).

Upon completion of the studies indicated above, you will be advised of additional follow-up actions required, as well as our conclusions regarding

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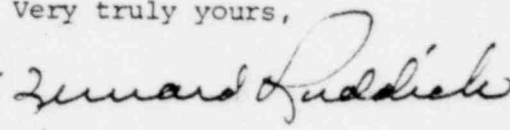
Mr. W.C. Seidle

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the safety significance and reportability of this matter. In the event this item cannot be satisfactorily resolved by June 1st, an additional interim status report will be prepared and forwarded to you.

Very truly yours,

for 
Glenn L. Koester
Vice President - Operations

GLK:bb