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REGULATORY DOCISET FU



Dear Mr. Stolz:

In response to your letter of June 25, 1976, we have completed a review of our Operational Quality Assurance Program and have compared our Program to the requirements of ANSI N18.7-1976.

The results of our review and comparison are summarized in the attached table and notes. The table cross-references each section of the ANSI N18.7-1976 to the appropriate sections of the FSAR wherein discussion is provided. In some cases, the current FSAR descriptions are not sufficient to allow your staff to determine compliance/non-compliance to the ANSI N18.7-1976. In these cases, we have provided additional discussion via the notes referenced in the table. The notes include in some instances proposed FSAR revisions which would document additional information describing how our Program addresses various requirements. In other instances the notes provide clarifying commentary which do not call for an FSAR revision. Where the word "none" appears in the table, the referenced material is sufficient.

9670

Mr. John F. Stolz - 2 - September 22, 1976

If, upon completion of your review of this transmittal, you feel that our response is adequate, please advise. We will then update the FSAR accordingly and implement the proposed program changes.

Very truly yours,

qual

J. T. Rodgeve Asst. Vice President

JTR/iw Attachments.

## COMPARISON ANALYSIS

ANSI N18.7-1976 REFERENCE SECTION

## FPC FSAR REFERENCE SECTION

## NOTE REFERENCE TO DETAILED COMMENT

1.	1.7.6.7.1 and 13.4	1
2.	None	2
3.1	1.7.6.7.1 (last paragraph, Page 1-58g)	3
3.2	1.7.5.1.1. 1.7.5.1.2. 1.7.6.7.1.a. and	지금 것 다 많은 모님
	1.7.6.7.1.b	None
3 3	176716	None
3 4 1	12.1	None
3 4 2	Appendix 12 A Sections 12 1 3 12 2 A	none
3.4.2	and 1 7 6 7 1	None
A 1	17671 x 17673 1767 A and	none
	1 7 6 7 5	None
1 2	1.7.0.7.5	None
4.2	17672 1767A and 17675	A
4.3.1	1.7.0.7.5, 1.7.0.7.4, and 1.7.0.7.5	Å
4.3.2	1.7.0.7.0 1.7.6.7.2 1.7.6.7.4 1.7.6.7.6 and	아이가 빠지 거야? 가지?
4.3.3	1.7.0.7.3, 1.7.0.7.4, 1.7.0.7.5, and	1
	1./.0./.1.q	4
4.3.4	12.0 and 1.7.0.7.5	4
4.4		None
4.5	1.7.b.7.1.r, 12.1.1, and 12.b	4
5.1	1./	None
5.2	12.1.2 and 1.7.6.7.2	None
5.2.1	12.1.2 and 1.7.6.7.1.a	5
5.2.2	1.7.6.7.1.e and 12.4	6
5.2.3	12 (all)	None
5.2.4	12.4	7
5.2.5	12.4	8
5.2.6	12.4	9
5.2.7	12.1.1, 1.7.6.7.1.r and 1.7.6.7.2	10
5.2.7.1	1.7.6.7.1.j, 1.7.6.7.1.k, and 1.7.6.7.1	
	(Page 1-59)	10,11
5.2.7.2	12.1.1, 1.7.6.7.1.r, and 1.7.6.7.2	10
5.2.8	12.4	12
5.2.9	12.3	None
5.2.10	12.4	13
5.2.11	1.7.6.7.1.p and 12.7	None
5.2.12	1.7.6.7.1.g	14
5.2.13	1.7.6.7.1.g	15
5.2.13.1	1.7.6.7.1.d, 1.7.6.7.1 (Page 1-58g, last	
	paragraph)	None
5.2.13.2	1.7.6.7.1.g	None
5.2.13.3	-1.7.6.7.1.h	None
5.2.13.4	1.7.6.7.1.m	16
5.2.14	1.7.6.7.1.0 and 1.7.6.7.2	None
5.2.15	1.7.6.7.1.f and 12.4	None
5.2.16	1.7.6.7.1.L	17

5.2.17	1.7.6.7.2.b and 1.7.6.7.1.j	None
5.2.18	1.7.6.7.1.j	None
5.2.19	1.7.6.7.1.k and 13.(all)	None
5.2.19.1	13.4	None
5.2.19.2	13.4	None
5.2.19.3	1.7.6.7.1.k	None
5.3	1.7.6.7.1.e and 12.4	None
6.	Not Applicable	18

#### NOTES

1. The last sentence in Section 1.7.6.7.1 which currently reads:

"Operation responsibility begins with commerical operation and extends throughout the life of the plant."

will be amended to read:

> "Operation responsibility begins with commencement of fuel loading and ends with plant decommissioning."

The transition of responsibilities for activities involving pre-critical and postcritical testing are clearly defined in the referenced FSAR sections.

- 2. Our review of ANSI N18.7-1976 was performed bearing in mind the definitions contained in Section 2. therein.
- 3. We have established an administrative controls and quality assurance program which shall be in effect at all times during the operational phase. We believe that our program does comply with or meets the intent of the subject standard.
- 4. The Nuclear General Review Committee does meet the requirements of ANSI N18.7-1976 for an independent review and audit program. A complete description is contained in the Committee Charter, which is available for NRC review.
- 5. The following de. cription will be added to FSAR Section 1.7.6.7.1.a. (This material is included in plant procedures.)

"The person charged with the duty of reactor operation has the authoricy and responsibility for shutting the reactor down to a safe condition by approved procedures when he determines that the safety of the reactor/plant is in jeopardy or when operating parameters exceed any of the reactor protection set points and automatic shutdown does not occur. Upon determination of shutdown, he shall notify the Shift Supervisor in charge who will insure that the plant is shut down in safe condition by approved procedures.

The Shift Supervisor in charge has the responsibility to notify the Operations Supervisor or person on-call and together they determine the circumstances, analyze the cause, and determine that operations can proceed safely before the reactor is returned to power after a trip or unscheduled or unexplained power reduction. Approval to take the reactor critical following a trip rests with the Operations Supervisor or person on-call, but the actual startup shall be authorized by the Shift Supervisor in charge and the authorization shall be documented in the Operator's Log.

The Shift Supervisor in charge has responsibility to be present at the plant and to provide direction for returning the reactor to criticality or power following a trip or unscheduled or unexplained power reduction.

All persons charged with the duty of operating the plant have the responsibility to:

- a. Believe and respond conservatively to instrument indications unless they are proven to be incorrect by instrument channel check or instrument channel test.
- b. Adhere to Technical Specifications.

The Shift Supervisor in charge and the Operations Supervisor have the responsibility to review routine operating data to assure safe operations."

 The following description will be added to FSAR Section 1.7.6.7.1.e. (This material is included in plant procedures.)

"Written procedures shall be strictly adhered to in all matters relating to nuclear safety. In order to properly document that procedural steps are verified as required, a check-off list system has been developed. Each step of the procedure requiring verification is included on the check-off list. The completed check-off list is signed and dated by the person performing the procedure, then the responsible supervisor approves and dates the completed check-off list.

Data or calculation sheets which are portions of a procedure are completed while the task is being performed. Each data sheet is initialed and dated by the person taking the data and approved by the responsible supervisor.

While performing Operations and Maintenance Procedures, the written procedure shall be present and followed step systep.

Emergency and Abnormal Procedures provide a guide to operators during potential emergencies or off-normal conditions which could conceivably degenerate into true emergencies. Since emergencies may not follow anticipated patterns, the operator action will be dictated by the situation. When immediate operator action is required to prevent or mitigate the consequences of a serious condition,

this action shall be implemented promptly. When the unit is stable, the appropriate Emergency or Abnormal Procedure(s) is reviewed to assure that all necessary steps were taken and the "Follow-Up Action" portion of the procedure performed with the written procedure(s) present.

Other procedures may contain routine procedural action that is frequently repeated and does not require that the procedure be present. It is required, however, that the persons performing these procedures be familiar with their content and adhere to the procedure requirements. If any doubt exists concerning an individual's ability to perform the procedure, he is required to have a copy of the procedure present.

Temporary minor changes to a procedure which do not change the intent of the original procedure and which are not safety-related are permitted only on documented approval of two appropriate supervisors, one of whom holds a Senior Reactor Operator's License. This action must be followed by a subsequent review by the Plant Review Committee and Nuclear Plant Superintendent within fourteen days of implementation. Procedures are provided for executing temporary changes to procedures including provisions for documentation, approval and subsequent formal incorporation."

7. The following paragraph will be added to FSAR Section 12.4, page 12-15.

"Short term instructions are any miscellaneous instructions that may be required and, when needed, shall be used for routine maintenance and personnel instruction. Short-term instructions are documented, serially numbered and shall automatically expire in 90 days if not previously canceled. It is the Shift Supervisor's responsibility to review, audit, and maintain the short-term instructions in a current and up-to-date condition. If it is necessary to continue the short-term instruction, it shall be reissued using a new document number. Short-term instructions shall not be amended. If a change is necessary to an issued short-term instruction, it shall be canceled and reissued in its correct form. The shortterm instruction is immediately effective when signed by a Shift Supervisor or the Operations Supervisor and official instruction removal occurs also with the initials of a Shift Supervisor or the Operations Supervisor. Canceled short-term instructions shall be destroyed."

8. The use of Temporary Procedures would be in conflict with Standardized Technical Specifications which do not provide for their use.

9. The following paragraph will be inserted in FSAR Section 12.4.

"Procedures are provided for control of equipment, as necessary, to maintain personnel and reactor safety and to avoid unauthorized operation of equipment. Release of installed plant systems for maintenance activities is done through plant equipment clearance procedures."

A clearance is permission to work on a mechanical or electrical system or component after the system or component has been isolated and is properly tagged and accepted. Only the Shift Supervisor can authorize clearances. Clearances are issued through the control center and documented. The person accepting the clearance is responsible for the safety of all persons working under his clearance. A clearance cannot be transferred from person to person. Acceptance of a clearance means personally

checking the system to assure the proper valving, switching and tagging have been performed. Acceptance signature is required. The Shift Supervisor is authorized to declare a system ready for service. Declaration is by his signature. A system ready for service requires that all clearances have been released, all tags removed, and all breakers, switches, valves, etc. are in their required line up.

10. Under Section 5.2.7 in ANSI N18.7-1976, on page 11, several standards are referenced as containing "useful guidance" for modification activities and "shall be applied" during the operational phase for those activities comparable in nature and extent to related activities occuring during initial plant design and construction.

FPC has developed maintenance and modification procedures and instructions in the lower tier documents to the FSAR that adequately meet the intent of this section. Exception is taken in acceptance of the mandatory application of the referenced standards in total. Review is made of the applicable standard revision when specific problem areas arise and ability to comply is confirmed or an alternate standard employed.

In addition, FSAR Section 1.7.6.7.1, page 1-59, paragraph 4 will be revised to read as follows:

"Maintenance or modifications which may affect safety-related structures, systems, or components shall be performed in a manner to ensure quality at least equivalent to that specified in original design bases and requirements, materials specifications, and inspection requirements. Maintenance or modifications of equipment shall be preplanned and performed in accordance with written procedures, documented instructions or drawings appropriate to the circumstances which conform to applicable codes, standards, specifications, and criteria."

- 11. Section 5.2.7.1 of ANSI N18.7-1976 requires a preventive maintenance program. FPC includes in its inservice inspection and surveillance program activities which we feel would constitute an acceptable preventive maintenance program for safety-related structures, systems and components. The basis for this program will be provided in the Technical Specifications which will be issued as Appendix A to the operating license. The implementation of this program will be through written, approved plant surveillance procedures.
- 12. Section 5.2.8 of ANSI N18.7-1976 defines the need for a surveillance testing and inspection program. FPC has developed a comprehensive surveillance testing and inspection program as outlined in the draft Technical Specifications and the test program identified in Section 1.7.6.7.k of the FSAR. Additionally, for clarification in this area and similar areas, Section 12.4 of the FSAR will be amended as follows:

Delete:

"The following is a list of the titles of all safety-related procedures. Additional procedures will be developed as required."

Add:

"The following is a representative list of the titles of key safety-related - procedures and functional support areas as defined in the Plant Operations Quality Assurance Manual."

Revise:

The listing on FSAR Pages 12-16 through 12-20 to reflect updated key safetyrelated procedures. (New list follows.)

#### OPERATING PROCEDURES

LIMITS AND PRECAUTIONS

PLANT LIMITS AND PRECAUTIONS PLANT SET POINTS

POOR ORIGINAL

PLANT OPERATING PROCEDURES

FLANT FRECRITICAL CHECK PLANT HEATUP (COLD TO HOT SHUTDOWN) PLANT STARTUP (HOT SHUTDOWN TO MINIMUM LOAD) POWER OPERATION HYDROGEN ADDITION AND DEGASIFICATION RECOVERY FROM REACTOR TRIP FIRE FROTECTION SYSTEMS TURBINE GENERATOR SHUTDOWN PLANT COOLDOWN REACTOR STARTUP REACTOR SHUTDOWN

REACTOR COOLANT SYSTEM OPERATING PROCEDURES

FILLING AND VENTING THE RC SYSTEM RC FUMP OPERATION DRAINING AND NITROGEN BLANKETING OF THE RC SYSTEM SOLUBLE FOISON CONCENTRATION CONTROL REACTIVITY BALANCE CALCULATIONS

AUXILIARY SYSTEM OPERATING FROCEDURES

CORE FLOODING SYSTEM MAKEUP AND PURIFICATION SYSTEM CHEMICAL ADDITION DECAY HEAT REMOVAL SYSTEM REACTOR BUILDING SPRAY SYSTEM SFENT FUEL COOLING SYSTEM LIQUID WASTE DISPOSAL SYSTEM NUCLEAR SERVICES COOLING SYSTEM PLANT VENTILATION SYSTEMS SECONDARY SERVICES CLOSED CYCLE COOLING SYSTEM INSTRUMENT AND HOUSE SERVICE AIR SYSTEMS WASTE GAS DISPOSAL SYSTEM WASTE DRUMMING SYSTEM NITROGEN AND HYDROGEN SYSTEMS DOMESTIC WATER SUPPLY SYSTEM CONTAINMENT OPERATING PROCEDURE DEMINERALICED WATER SYSTEM

#### OPERATING PROCEDURES (continued)

INSTRUMENTATION AND CONTROL SYSTEM OPERATING PROCEDURES

REACTOR NON-NUCLEAR INSTRUMENTATION CONTROL ROD DRIVE SYSTEM PLANT COMPUTER SYSTEM INTEGRATED CONTROL SYSTEM

#### STEAM SYSTEM OPERATING PROCEDURES

CONDENSATE DEMINERALIZER SYSTEM TURBO-GENERATOR OIL SYSTEM CONDENSATE SYSTEM CIRCULATING WATER SYSTEM FEEDWATER SYSTEM AUXILIARY STEAM SYSTEM CONDENSER VACUUM SYSTEM OTSG SECONDARY FILL, DRAIN, AND LAYUP CHLORINATION SYSTEM

POOR ORIGINAL

ELECTRICAL SYSTEMS CPERATING PROCEDURES

GENERATOR GAS SYSTEM PLANT DISTRIBUTION SYSTEM COMMUNICATIONS SYSTEM EMERGENCY POWER-DC SYSTEM EMERGENCY POWER-DIESEL GENERATORS RECEIVING DIESEL FUEL

#### EMERGENCY PROCEDURES

#### EMERGENCY PLAN IMPLEMENTING PROCEDURES

DUTIES OF AN INDIVIDUAL WHO DISCOVERS AN EMERGENCY DUTIES OF THE EMERGENCY COORDINATOR CLASSIFICATION OF EMERGENCIES AND CRITERIA FOR EVACUATION DOSE ASSESSMENT BY USE OF METEOROLOGICAL OVERLAYS EVACUATION AND PERSONNEL ACCOUNTABILITY OF CR-3 PERSONNEL EMERGENCY PLAN ROSTER AND NOTIFICATION REPORTING REQUIREMENTS ON EMERGENCIES DUTIES OF THE RADIATION EMERGENCY TEAM RE-ENTRY PROCEDURE DUTIES OF THE ENVIRONMENTAL SURVEY TEAM DUTIES OF THE SECURITY OFFICER DUTIES OF THE PLANT GUARDS MEDICAL EMERGENCY PROCEDURES DUTIES OF FOSSIL PLANT PERSONNEL POOR ORIGINAL POST-ACCIDENT REACTOR BUILDING PURGING DUTIES OF THE NUCLEAR PLANT FIRE BRIGADE

#### PLANT EMERGENCY PROCEDURES

UNIT BLACKOUT LOSS OF CONTAINMENT INTEGRITY LOSS OF RC FLOW/RC FUMP TRIP STEAM GENERATOR TUBE FAILURE STEAM SUPPLY SYSTEM RUPTURE LOSS OF RC/RC PRESSURE LOSS OF BORON (MODERATOR DILUTION) LOSS OF STEAM GENERATOR FEED EARTHQUAKE CRD MALFUNCTION ACTION FIRE LOSS OF DECAY HEAT REMOVAL SYSTEM FLANT SHUTDOWN FROM OUTSIDE CONTROL CENTER

#### ABNORMAL PROCEDURES

ABNORMAL 'PLANT PROCEDURES

EXPECTED TRANSIENTS ANNUNCIATOR ALARMS RADIATION MONITORING SYSTEM ALARMS VIOLENT WEATHER EXCESSIVE POWER IMBALANCE AND/OR QUADRANT POWER TILT LOAD REJECTION TURBINE TRIP LOSS OF CONDENSER VACUUM LOSS OF NUCLEAR SERVICES COOLING REACTOR TRIP REACTOR HIGH STARTUP RATE REACTOR COOLDOWN BY NATURAL CIRCULATION LOSS OF FEEDWATER CHEMICAL CONTROL LOSS OF RC MAKEUP PRESSURIZER SYSTEM ABNORMAL CONDITIONS LOSS OF OUT-OF-CORE FLUX INDICATION LOSS OF INSTRUMENT AIR

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#### MAINTENANCE PROCEDURES

REACTOR COOLANT SYSTEM MAINTENANCE

PRESSURIZER SPRAY HEAD REMOVAL AND REPLACEMENT PRESSURIZER RELIEF VALVE REMOVAL AND REPLACEMENT FREEZE SEAL FOR NUCLEAR PIPING SYSTEMS OTSG TUBE PLUGGING FILTER REMOVAL AND REPLACEMENT (LIQUID) ABSOLUTE AND CHARCOAL FILTER REMOVAL AND REPLACEMENT PRESSURIZER HEATER BUNDLE REMOVAL AND REPLACEMENT CONTROL ROD DRIVE HANDLING OTSG RELIEF VALVE REMOVAL AND REPLACEMENT OTSG MAINTENANCE VALVE PACKING PROCEDURE AND SPECIFICATIONS REACTOR INTERNALS VENT VALVE REMOVAL AND REPLACEMENT LUBRICATION SCHEDULE IN-CORE MONITOR FIFING REPAIR RC PUMP INSPECTION AND REPLACEMENT VALVE BONNET REMOVAL AND REINSTALLATION RELIEF VALVE REMOVAL, REPAIR, AND REINSTALLATION REMOVAL AND REPLACEMENT OF PRESSURE SEAL VALVE BONNETS PUMP REPACKING DISASSEMBLY AND REASSEMBLY OF FLANGED CONNECTIONS DISASSEMBLY AND REASSEMBLY OF NUCLEAR SERVICES CLOSED CYCLE COOLING FUMPS 1A. 1B. and 1C DISASSEMBLY AND REASSEMBLY OF EMERGENCY FEEDWATER FUMPS INSTALLATION OF FUEL TRANSFER TUBE COVERS DISASSEMBLY AND ASSEMBLY OF MAKEUF PUMPS REMOVAL AND REPLACEMENT OF RC PUMP HYDRAULIC SHOCK SUPPRESSORS PIPE SNUBBER MAINTENANCE

INSTRUMENTATION AND CONTROL SYSTEM MAINTENANCE

NUCLEAR DETECTOR REMOVAL AND REPLACEMENT

ELECTRICAL SYSTEM MAINTENANCE

DC SYSTEM MAINTENANCE

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#### REFUELING PROCEDURES

2

REFUELING OPERATION SEQUENCE OF EVENTS

REFUELING OPERATION SEQUENCE OF EVENTS INITIAL FUEL LOADING

PREPARATION FOR REFUELING

PREPARATION FOR REFUELING FRESH FUEL UNPACKING, INSPECTION, AND STORAGE NEW CONTROL COMPONENT RECEIPT, INSPECTION, AND STORAGE FILL AND DRAIN OF THE FUEL TRANSFER CANAL

CLOSURE HEAD REMOVAL AND REPLACEMENT

REACTOR VESSEL CLOSURE HEAD REMOVAL AND REPLACEMENT REACTOR VESSEL CLOSURE HEAD STUD REMOVAL AND REPLACEMENT HEAD SEAL LEAK TEST CANAL SEAL PLATE REMOVAL AND REPLACEMENT

REACTOR INTERNALS REMOVAL AND REPLACEMENT

REACTOR INTERNALS REMOVAL AND REPLACEMENT

FUEL HANDLING

FUEL HANDLING EQUIPMENT OPERATIONS

IN-CORE MONITOR HANDLING AND REPLACEMENT

IN-CORE MONITOR HANDLING

FAILED FUEL IDENTIFICATION AND HANDLING

FAILED FUEL IDENTIFICATION FAILED FUEL HANDLING

NEUTRON SOURCE HANDLING

PRIMARY NEUTRON SOURCE RECEIPT, INSPECTION, STORAGE AND INSTALLATION

POOR ORIGINAL

SPENT FUEL SHIPMENT

SFENT FUEL HANDLING

#### CHEMISTRY AND RADIATION PROTECTION PROCEDURES

RADIOLOGICAL CONTROL PROCEDURES

RADIATION PROTECTION MANUAL RESPIRATORY EQUIPMENT MANUAL DECONTAMINATION OF PERSONNEL, AREAS, AND EQUIPMENT RADIOACTIVE LIQUID WASTE RELEASE PERMIT PROCEDURE RADIOACTIVE AIRBORNE RELEASE PERMIT PROCEDURE RADIATION WORK PERMIT PROCEDURE STANDING RADIATION WORK PERMIT PROCEDURE RADIATION PROTECTION INSTRUCTIONS MANUAL

CHEMISTRY FROCEDURES

CHEMISTRY AND RADIOCHEMISTRY PROCEDURES

GUIDELINES FOR CHEMICAL ANALYSIS GUIDELINES FOR RADIOCHEMICAL ANALYSIS CHEMISTRY SAMPLING PROCEDURES

#### COMPLIANCE PROCEDURES

#### COMPLIANCE PROCEDURES

QUALITY SYSTEM MATERIALS PURCHASE QUALITY SYSTEM MATERIALS RECEIPT, INSPECTION, AND STORAGE WELDING ROD CONTROL WELDING PROCEDURE QUALIFICATION WELDER QUALIFICATION DOCUMENTATION FOR GA SYSTEM WELDING TEST INSTRUMENTS, STANDARDS, AND CALIBRATICN CONTROL NON-DESTRUCTIVE EXAMINATION NON-DESTRUCTIVE EXAMINATION PERSONNEL QUALIFICATION PROCEDURE FOR CONFLIANCE AUDIT OF FLANT FUNCTIONS PROCEDURE FOR IMPLEMENTATION & THE MASTER SURVEILLANCE PROGRAM ACTIVITY REVIEW OF PLANT FUNCTIONS FROCEDURE FOR HANDLING DISCREPANCIES AND CORRECTIVE ACTION PROCEDURE FOR CONTROL OF PERMANENT MODIFICATIONS, TEMPORAPY MODIFICATIONS, AND DEVIATIONS IN-PLANT EQUIPMENT CLEARANCE AND SWITCHING ORDERS STANDARD CLEANLINESS SPECIFICATIONS FIRE PREVENTION WORK FERMIT

POOR ORIGINAL

#### SURVEILLANCE PROCEDURES

#### TECHNICAL SUPPORT

COMPARISON OF OVERALL CORE REACTIVITY BALANCE TO PREDICTED VALUES MODERATOR TEMPERATURE COEFFICIENT DETERMINATION AT 300 PPM BORON

OR ORIGINAL

CONTROL ROD DROP TIME TESTS

HOT CHANNEL FACTORS CALCULATIONS

CORE PERFORMANCE PROGRAM

REACTOR FROTECTIVE SYSTEM FUNCTIONAL TESTING CALIBRATION OF THE REACTOR PROTECTION SYSTEM NUCLEAR INSTRUMENTATION CALIBRATION ENGINEERED SAFEGUARDS MONTHLY FUNCTIONAL TESTS

STEAM LINE RUPTURE PRESSURE SWITCH CALIBRATION

ENGINEERED SAFEGUARDS CHANNEL CALIBRATION

ENGINEERED SAFEGUARDS ACTUATION SYSTEM RESPONSE TIME

IN-CORE NEUTRON DETECTOR SYSTEM CALIBRATION

FUNCTIONAL TESTING OF THE TRIAXIAL TIME-HISTORY ACCELOGRAPHS AND TRIAXIAL SEISMIC SWITCH AND CALIBRATION OF THE TRIAXIAL

SEISMIC SWITCH CALIBRATION OF THE TRIAXIAL TIME-HISTORY ACCELOGRAPHS AND TRIAXIAL PEAK ACCELOGRAPHS

MFTEOROLOGICAL MONITORING INDTRUMENTATION CALIBRATION REMOTE SHUTDOWN INSTRUMENTATION CALIBRATION POST-ACCIDENT MONITORING INSTRUMENTATION CALIBRATION PLANT INSTRUMENT CALIBRATION PRESSURIZER LEVEL INSTRUMENTATION CALIBRATION ONCE-THROUGH STEAM GENERATOR INSERVICE INSPECTION CONTAINMENT SUMP LEVEL AND FLOW MONITORING SYSTEM CALIBRATION CONTAINMENT LEAKAGE TEST-TYPE "A", INCLUDING LINEP PLATE CONTAINMENT LEAKAGE TEST-TYPE "B" CONTAINMENT AIR LOCK TEST (SEMIANNUAL) CONTAINMENT SPRAY SYSTEM SPRAY NOZZLE FLOW CHECK NUCLEAR OVERPOWER TRIP SET POINTS DURING PHYSICS TESTING ZERO POWFR PHYSICS TESTING: SOURCE RANGE, INTERMEDIATE RANGE,

AND NUCLEAR OVERPOWER CHANNEL FUNCTIONAL TESTS FRODUCTION DEFARTMENT MONTHLY REPORT INSTRUMENTATION FUNCTIONAL TESTS DURING REFUELING OPERATIONS STARTUF FEEDWATER VALVES AIR ACCUMULATORS LEAK TEST REACTOR VESSEL AND INTERNALS INSPECTION

#### OPERATIONS

OPERATING DAILY SURVEILLANCE LOG SHUTDOWN DAILY SURVEILLANCE LOG HEAT BALANCE CALCULATIONS WEEKLY OPERABILITY VERIFICATION OF BORON INJECTION SOURCES, PUMPS, AND FLOW PATHS POWER DISTRIBUTION BREAKER ALIGNMENT AND POWER AVAILABILITY VERIFICATION SURVEILLANCE PROCEDURES (continued)

# POOR ORIGINAL

OPERATIONS (continued)

CABLE TUNNEL SUMP PUMPS OPERABILITY VERIFICATION FIRE AND EVALUATION ALARMS MISCELLANEOUS MONTHLY CHECKS MONTHLY FEEDWATER ISCLATION FUNCTIONAL TESTS CONTROL ROD EXERCISES RADIATION MONITORING INSTRUMENTATION FUNCTIONAL TEST TRIAXIAL TIME-HISTORY ACCELOGRAPH CHANNEL CHECK REMOTE SHUTDOWN INSTRUMENTATION CHANNEL CHECK MONTHLY CORE FLOOD TANK OPERABILITY VERIFICATION DEMONSTRATION OF ECCS AND BORATION SYSTEMS OPERABILITY MONTHLY CONTAINMENT INTEGRITY CHECK REACTOR BUILDING SPRAY SYSTEM VALVES CHECK MAIN STEAM ISOLATION VALVES OPERABILITY NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM OPERABILITY AUXILIARY BUILDING VENTILATION EXHAUST SYSTEM MONTHLY TEST CONTAINMENT PENETRATIONS WEEKLY CHECK DURING REFUELING OPERATIONS CONTAINMENT PRAY SYSTEM OPERABILITY DEMONSTRATION CONTAINMENT COOLING SYSTEM OPERABILITY DEMONSTRATION AUXILIARY FEEDWATER SYSTEM OPERABILITY DEMONSTRATION DECAY HEAT CLOSED CYCLE COOLING OPERABILITY NUCLEAR SERVICES SEAWATER SYSTEM DECAY HEAT SEAWATER SYSTEM CONTROL ROOM EMERGENCY VENTILATION SYSTEM MONTHLY TEST EMERGENCY DIESEL FUEL OIL QUALITY AND DIESEL GENERATOR MONTHLY TEST MONTHLY TURBINE GENERATO, CHECKS FEEDWATER JURBINE VALVES TEST FIRE FROTECTION SYSTEM TESTS HOSE STATION INVENTORY AND HYDRANT OPERABILITY TEST QUARTERLY CYCLING OF POWER-OPERATED CONTAINMENT ISOLATION VALVES SPRAY ADDITIVE SYSTEM SODIUM HYDROXIDE AND SODIUM THIOSULFATE TAUKS ANALYSIS STARTUP SURVEILLANCE LOG SHUTDOWN MARGIN PHYSICS TESTING CONTROL ROD PROGRAMMING VERIFICATION CORE FLOODING SYSTEM ISOLATION VALVES ALAFMS ACTUATION DECAY HEAT REMOVAL SYSTEM AUTOMATIC CLOSURF AND INTERLOCK VERI-FICATION CORE FLOCDING SYSTEM CHECK VALVE OFERATION DEMONSTRATION REFUELING OPERATIONS DAILY DATA REQUIREMENTS FEEDWATER BOOSTER PUMPS TEST CYCLING DEMONSTRATION OF NORMALLY UNTESTABLE POWER-OPERATED VALVES FIRE FROTECTION, DELUGE, AND SPRINKLER SYSTEMS TEST ECCS AND CONTAINMENT SPRAY SYSTEM LEAK RATE TEST REFUELING INTERVAL CYCLING OF MANUAL CONTAINMENT ISOLATICE VALVES TRANSFER FROM PREFERRED OFF-SITE FOWER SOURCE TO ALTERNATE OFF-SITE SOURCE AND RETURN TO PREFERRED OFF-SITE SOURCE

#### SURVEILLANCE PROCEDURES (continued)

# POOR ORIGINAL

OPERATIONS (continued)

EMERGENCY FEEDWATER AUTOMATIC ACTUATION REFUELING INTERVAL INTEGRATED PLANT RESPONSE TO ES ACTUATION TURBINE OVERSPEED TRIP RC SYSTEM HEATUP AND COOLDOWN SURVEILLANCE STEAM GENERATORS SPECIAL TEST LOG PHYSICS TESTING HOURLY SURVEILLANCE LOG CONTROL ROD DRIVE PATCH PANEL ACCESS CONTROL EQUILIBRIUM XENON REACTIVITY POWER LEVEL CUT-OFF VERIFICATION OF INITIAL CONDITIONS PRIOR TO SPENT FUEL CASK HANDLING CONTAINMENT AIR LOCK SHUTDOWN MARGIN WITH INOPERABLE CONTROL ROD IN-CORE NEUTRON DETECTORS CHANNEL CHECK FUEL STORAGE POOL MISSILE SHIELDS UNIT STARTUP SURVEILLANCE PLAN UNIT SHUTDOWN SURVEILLANCE PLAN SPECIAL CONDITIONS SURVEILLANCE PLAN

#### ELECTRICAL MAINTENANCE

HALON FIRE PROTECTION SYSTEM CHECKS WEEKLY BATTERY CHECK (UNITS 1 & 2) QUARTERLY BATTERY CHECK (UNITS 1 & 2) BATTERY INSFECTION AND CHARGER TEST (UNITS 1 & 2) WEEKLY BATTERY CHECK QUARTERLY BATTERY CHECK STATION BATTERIES INSPECTION AND BATTERY CHARGER LOAD TEST BATTERY PERFORMANCE DISCHARGE TEST DEMONSTRATION OF AUXILIARY BUILDING OVERHEAD CRANE (FHCR-5) INTERLOC, OPERABILITY SPENT FUEL HANDLING BRIDGE INTERLOCK SURVEILLANCE REACTOR BUILDING MAIN AND AUXILIARY (FHCR-1 AND FHCR-2) FUEL HANDLING BRIDGES ELECTRICAL INTERLOCK SURVEILLANCE

#### MECHANICAL MAINTENANCE

PRESSURICER CODE SAFETY VALVES TEST PROCEDURE FOR LOAD TESTING SLINGS EMERGENCY DIESEL GENERATOR ENGINE INSPECTION/MAINTENANCE MAIN STEAM CODE SAFETY VALVES TEST REACTOR BUILDING FUEL HANDLING BRIDGES (FHOR-1 & 2) LOAD TEST SPENT FUEL HANDLING BRIDGE LOAD SURVEILLANCE FROCEDURE FOR LOAD TESTING NEW FUEL ELEVATOR

# POOR ORIGINAL

#### SURVEILLANCE FROCEDURES (continued)

#### CHEMISTRY AND RADIATION PROTECTION

RADIATION MONITORING INSTRUMENTATION CALIBRATION REACTOR COOLANT SYSTEM DISSOLVED OXYGEN ANALYSIS REACTOR COOLANT SYSTEM CHLORIDE AND FLUORIDE ANALYSIS REACTOR COOLANT SYSTEM GROSS ACTIVITY DETERMINATION REACTOR COOLANT SYSTEM DOSE EQUIVALENT I-131 DETERMINATION REACTOR COOLANT SYSTEM E-BAR DETERMINATION REACTOR COOLANT SYSTEM ISOTOPIC ANALYSIS FOR IODINE INCLUDING I-131, I-133 AND I-135

SECONDARY COOLANT SYSTEM GROSS BETA ACTIVITY DETERMINATION SECONDARY COOLANT SYSTEM ISOTOPIC ANALYSIS FOR DOSE EQUIVALENT I-131 CONCENTRATION

HYDROGEN ANALYZER CALIBRATION SEALED SOURCE TESTING STORED SOURCE TESTING STARTUP SOURCE TESTING REACTOR COOLANT SYSTEM AND REFUELING CANAL BORON CONCENTRATION

#### ADMINISTRATION

FORTABLE FIRE EXTINGUISHER MONTHLY CHECK PORTABLE FIRE EXTINGUISHER YEARLY CHECK

MASTER SURVEILLANCE PLAN

DOCUMENT CONTROL PROCEDURES

#### DOCUMENT CONTROL PROCEDURES

DRAWING AND DOCUMENT CONTROL

SPECIAL NUCLEAR MATERIALS HANDLING AND ACCOUNTABILITY MANUAL

SNM HANDLING AND ACCOUNTING MANUAL

#### SECURITY PROCEDURES

#### SECURITY PROCEDURES

POOR ORIGINAL

INTRODUCTION TO GUARD FORCE OPERATIONS GUARD FORCE OPERATIONS PERSONNEL ACCESS CONTROLS IDENTIFICATION BADGE SYSTEM SECURITY KEY AND LOCK CONTROLS MATERIALS AND PACKAGE PASS SYSTEM SECURITY ALARMS AND TELEVISION SYSTEM OPERATIONS SECURITY DURING DECLARED EMERGENCIES SECURITY DURING BOMB OR OTHER OVERT THREATS SECURITY DURING CIVIL DISTURBANCES EMPLOYEE SECURITY INDOCTRINATION AND TRAINING AUDITING OF SECURITY PROCEDURES

#### .ADMINISTRATIVE INSTRUCTIONS

#### ADMINISTRATIVE INSTRUCTIONS

FACILITY ADMINISTRATIVE POLICIES ORGANIZATION AND RESPONSIBILITY PLANT REVIEW COMMITTEE CHARTER PLANT OPERATING QUALITY ASSURANCE MANUAL CONTROL DOCUMENT CONDUCT OF OPERATIONS CONDUCT OF MAINTENANCE CONDUCT OF CHEMISTRY AND RADIATION PROTECTION CONDUCT OF ADMINISTRATIVE SERVICES CONDUCT OF TECHNICAL SUPPORT CONDUCT OF COMPLIANCE SCOPES OF PROCEDURES 13. The following paragraph will be added to FSAR Section 12.4.

"Plant procedures for housekeeping and cleanliness control are provided which meet the intent of ANSI N-45.2.3-1973 and N45.2.1-1973".

14. The following paragraph will be added to FSAR Section 1.7.6.7.1.g.

"Plant records management controls are provided which meet the intent of ANSI N45.2.9-1974".

- 15. FPC to date has not received a copy of ANSI N45.2.13-1976. This standard is currently on order. No commitment can be made at this time due to the unavailability of the 1976 standard.
- 16. The following paragraph will be added to FSAR Section 1.7.6.7.1.m:

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"Handling, Storage, and Shipping procedures are provided which meet the intent of ANSI N45.2.2-1972."

17. The following paragraph will be added to FSAR Section 1.7.6.7.1.L:

"Calibration records and controls are provided which meet the intent of ANSI N45.2.4-1972."

