



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
230 PEACHTREE STREET, N. W. SUITE 818
ATLANTA, GEORGIA 30303

IE Inspection Report No. 50-302/76-12

Licensee: Florida Power Corporation
3201 34th Street, South
P. O. Box 14042
St. Petersburg, Florida 33733

Facility Name: Crystal River 3
Docket No.: 50-302
License No.: CPPR-51
Category: B1

Location: Crystal River, Florida

Type of License: B&W, PWR, 2452, Mwt

Type of Inspection: Routine, Announced

Dates of Inspection: June 28 - July 2, 1976

Dates of Previous Inspection: June 15-18, 1976

Inspectors: F. Jape Reactor Inspector
Reactor Projects Section No. 2
Reactor Operations and Nuclear
Support Branch

Accompanying Inspectors: None

Other Accompanying Personnel: None

Principal Inspector: Frank Jape
F. Jape, Reactor Inspector
Reactor Projects Section No. 2
Reactor Operations and Nuclear
Support Branch

7/13/76
Date

Reviewed By: R. C. Lewis
R. C. Lewis, Chief
Reactor Projects Section No. 2
Reactor Operations and Nuclear
Support Branch

7/13/76
Date

8002270703

SUMMARY OF FINDINGS

- I. Enforcement Matters
None
- II. Licensee Action on Previously Identified Enforcement Matters
Not inspected.
- III. New Unresolved Items
None
- IV. Status of Previously Identified Unresolved Items
75-8/3 Operability of Category I Hydraulic Shock and Sway Suppressors (IEB 75-05)

Suppressor inspection currently in progress during HFT. Item remains open pending completion of inspection requirements. (Details, paragraph 3.a)
- V. Unusual Occurrences
None
- VI. Other Significant Finding
None
- VII. Management Interview

A management interview was held on July 2, 1976, with J. Alberdi, and members of his staff. The inspection findings related to witnessing preoperational testing activities were discussed.

DETAILS

Prepared by:

Frank Jape
 F. Jape, Reactor Inspector
 Reactor Projects Section No. 2
 Reactor Operations and Nuclear
 Support Branch

7/15/76

Date

Dates of Inspection: June 28-July 2, 1976

Reviewed by:

R. C. Lewis
 R. C. Lewis, Chief
 Reactor Projects Section No. 2
 Reactor Operations and Nuclear
 Support Branch

Date

1. Individuals ContactedFlorida Power Corporation (FPC)

E. E. Froats - Manager, Site Surveillance
 R. A. Parker - Chief Nuclear Operator
 W. W. Surrency - Chief Nuclear Operator
 T. N. Mount - Nuclear Operator
 G. P. Hebb - Assistant Nuclear Operator
 J. F. Heilman - Assistant Nuclear Operator
 K. N. Stuart - Assistant Nuclear Operator
 W. E. Kemper - Assistant Nuclear Operator
 W. M. Mathews - Shift Supervisor
 W. R. Nichols - Operations Supervisor
 H. J. Eck - Chemistry/Radiation Technician
 J. R. Wright - Chemistry/Radiation Protection Engineer
 J. C. Hobbs, Jr. - Manager, Generation Testing
 A. P. Vogt - Testing Superintendent

J. A. Jones, Inc. (JAJ)

J. Shively - Calibration Technician

Babcock & Wilcox, Inc. (B&W)

J. J. Kelley - Shift Testing Superintendent
 J. R. Bohart - Startup Test Engineer
 W. R. Jacobs - Startup Test Engineer

NUS

B. Seaholts - Testing Engineer
 J. H. Greene - Startup Test Engineer

Volt

W. Mathis - System Test Engineer
T. Phelps - System Test Engineer
E. T. Childress - Startup Test Engineer

2. Witness of Integrated Hot Functional Testing

The inspector witnessed a portion of test procedure TP 7 1 600 0, "Hot Functional Testing." The primary coolant system was at a nominal 532°F and 2155 psig during periods of inspector observation.

In general, the inspector verified that the proper procedures were in use, test prerequisites were met, the required systems to support the ongoing testing were placed in service as prescribed by procedures, crew actions were timely, attentive and responsive, and the records being maintained were as prescribed by the test program guide.

Discussions with operators and testing personnel revealed that they were using the current revision of the procedure. The operators at the control panels in the control room and those in charge of operating equipment in other locations throughout the plant were observed to be attentive and responsive.

The chronological log was reviewed and found to contain detailed information concerning the conduct of testing. Log entries concerning the ongoing testing were found to be complete. The inspector had no adverse comments or questions regarding this record.

3. Witness of Preoperational Testing

Portions of the following tests were witnessed by the inspector. The performance of each test was evaluated against the requirements of ANSI N18.7 - 1972, Rev. 1, Section 5.3.10, "Test and Inspection Procedures," ANSI N45.2 - 1971 Section 12, "Test Control," and FSAR Chapter 13, "Initial Tests and Operations." There were no discrepancies identified by this inspector.

a. TP 600-14, "Pipe and Component Hanger Hot Inspection Test"

The inspector observed licensee personnel recording measurements for pipe hangers and suppressors in the reactor building at normal operating pressure and temperature. In general, these data appeared to agree with prescribed values. At least one suppressor was observed to be out-of-service due to its physical arrangement. A change request for rearranging this suppressor has been instituted.

Licensee personnel were also observed making measurements of high energy lines and auxiliary piping in the auxiliary and turbine buildings. Sections of the feedwater lines were observed to be vibrating and had been lashed down with cables. Permanent hangers are planned to be installed for these lines following cooldown.

Measurements and data obtained for PT 600-14 also fulfill the licensee's commitments to IEB 75-05, "Operability of Category I Hydraulic Shock and Sway Suppressors." The required surveillance procedures for periodic inspection of snubbers and restraints have not been issued. This subject has been previously identified as unresolved item 75-8/3. The unresolved item will remain open pending issuance of an acceptable surveillance procedure and review and acceptance of data from TP 600-14.

b. TP 330-3, "CDR Functional Test," and TP 600-17, "CDR Operational Test"

The inspector observed the final portion of TP 330-3 being run. Each rod was operated individually during this test and accuracy of position indicators (PI) were verified. The 12 PI tubes reported defective in IE Report 50-302/76-7 have been repaired and rechecked. At the completion of this test all PI tubes were operational.

The objective of TP 600-17 was to operate the rod system in its normal configuration while checking the asymmetric fault indication, group position indication and sequence monitors. The inspector witnessed the initial part of this procedure.

A major change to the procedure was made prior to initiating the test. The change was processed as prescribed by Test Program Guide 1.1, Section 2.7.1.1. The change involved clarification of acceptance criteria. The inspector also verified the initial test conditions were as required by the procedure.

c. TP 600-23, "RPS Functional Test"

Checkout of portions of the reactor protective system (RPS) was witnessed by the inspector. Features such as system logic, channel bypass key switch, shutdown bypass key switch, loss of vital bus power source and simulated trip conditions for the protective functions were verified. The system performed as described in Section 7.1 of the FSAR.

d. TP 600-3, "Soluble Poison Concentration Control"

The ability to adjust boron concentration in the reactor coolant system was witnessed by the inspector. In addition, the laboratory technique for analyzing boron concentration was observed.

e. TP 600-12, "Pressurizer Test"

The portion of this test which dealt with checking and resetting the pressurizer relief valves at operating temperature was witnessed. RCV-8F required resetting since on initial check the popping pressure was out-of-allowable tolerance. The valve was reset and retested in place, by using a hydroset.

The calibration records for the test gauge and hydro pump used for performing these tests were in conformance with the requirements of Generation Test Procedure 1.3.