

LICENSEE EVENT REPORT

CONTROL BLOCK:           

[PLEASE PRINT ALL REQUIRED INFORMATION]

LICENSEE NAME: F L C R P 3      LICENSE NUMBER: 00-00000-00      LICENSE TYPE: 41111      EVENT TYPE: 01

CONT:                 CATEGORY: -      REPORT TYPE: T      REPORT SOURCE: I      DOCKET NUMBER: 050-0302      EVENT DATE: 061477      REPORT DATE: 062077

EVENT DESCRIPTION

02 Subsequent to introduction of NaOH into the RCS as reported in LER-77-17, B&W evaluation  
03 of this event indicated the possibility of an unreviewed safety question as defined by  
04 10 CFR50.59(3) in that the untermiated injection of the NaOH tank contents into the  
05 RCS could result in core criticality with all rods inserted. Redundancy N/A. This  
06 event has not occurred to this date. Administrative and physical controls to preclude

SYSTEM CODE: C B      CAUSE CODE: B      COMPONENT CODE: X X X X X X      PRIME COMPONENT SUPPLIER: A      COMPONENT MANUFACTURER: Z Z Z Z      VIOLATION: N      (LER 77-52)

CAUSE DESCRIPTION

08 The possibility of moderator dilution by the untermiated injection of the entire  
09 contents of the NaOH tank into the Reactor Coolant while in the decay heat removal  
10 cycle.

FACILITY STATUS: Z      % POWER: 000      OTHER STATUS: NA      METHOD OF DISCOVERY: Z      DISCOVERY DESCRIPTION: NA

FORM OF ACTIVITY RELEASED: Z      CONTENT OF RELEASE: Z      AMOUNT OF ACTIVITY: NA      LOCATION OF RELEASE: NA

PERSONNEL EXPOSURES

13 NUMBER: 000      TYPE: Z      DESCRIPTION: NA

PERSONNEL INJURIES

14 NUMBER: 000      DESCRIPTION: NA

OFFSITE CONSEQUENCES

15 NA

LOSS OR DAMAGE TO FACILITY

16 TYPE: Z      DESCRIPTION: NA

PUBLICITY

17 NA

ADDITIONAL FACTORS

18 this occurrence have been provided. 8002270643

19 See attached Supplementary Information

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SUPPLEMENTARY INFORMATION

1. Report No.: 50-302/77-52
2. Facility: Crystal River Unit #3
3. Report Date: 20 June 1977
4. Occurrence Date: 14 June 1977
5. Identification of Occurrence:

Possible unreviewed safety question as defined by 10 CFR50.59(b) concerning uninterminated sodium hydroxide dilution of Reactor Coolant System.

6. Conditions Prior to Occurrence:

NA as event has not occurred to this date.

7. Description of Occurrence:

As reported on 1 March 1977 (LER 77-17) approximately 600 gallons of NaOH was introduced into the Reactor Coolant System via the Decay Heat Removal System. Subsequent to this event, an evaluation by Babcock and Wilcox, concerning the uninterminated injection of the entire contents of the sodium hydroxide tank into the Reactor Coolant System, reveals that the possibility of this occurrence may constitute an unreviewed safety question. The results of this evaluation indicate that a situation could exist where an uninterminated moderator dilution accident could occur that is not bounded by the CR-3 FSAR Chapter 14 analysis.

8. Designation of Apparent Cause:

The injection of the entire NaOH tank contents into the RCS as described in the B&W evaluation could cause the core to go critical.

9. Analysis of Occurrence:

Should this event ever occur, there is the possibility of the Reactor going critical with all rods inserted.

10. Corrective Action:

Precluding an occurrence of this nature includes the following administratively imposed safeguards:

- A. Closing manual isolation valves in the NaOH supply lines and "racking out" the breakers to the motor operated isolation valves in the NaOH supply lines before starting a DH pump.
- B. Verifying manual isolation valves are closed before testing the motor operated valves in the NaOH supply lines.
- C. Only testing the motor operated valves at a refueling interval.

Corrective Action (Cont'd)

Florida Power Corporation is presently evaluating the chemical additive system for CR #3 to determine what permanent modifications are required as a result of eliminating the sodium thiosulfate from the system. This evaluation is being performed in accordance with Condition 2.C(4) of Amendment No. 1 to the CR #3 Operating License and will be submitted to the Commission for review and approval on or before September 3, 1977. It is our intent to include this unreviewed safety question as part of our overall evaluation of the Chemical Additive System. Appropriate revisions to the CR #3 FSAR will be submitted to the Commission following the completion of our evaluation.