

South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

November 4, 2019 NOC-AE-19003691 10 CFR 50.73

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Attention: Document Control Desk U.S. Nuclear Regulatory Commission Washington, DC 20555-0001

> South Texas Project Unit 2 Docket No. STN 50-499 Licensee Event Report 2019-001-00 Equipment Clearance Order Error Leads to Loss of Primary Containment Integrity

Pursuant to reporting requirements of 10 CFR 50.73(a)(2)(i)(B), 10 CFR 50.73(a)(2)(v)(C), and 10 CFR 50.73(a)(2)(v)(D), STP Nuclear Operating Company hereby submits the attached South Texas Project Unit 2 Licensee Event Report 2019-001-00.

The event did not have an adverse effect on the health and safety of the public.

There are no commitments in this submittal.

If there are any questions, please contact Tim Hammons at 361-972-7347 or me at 361-972-7888.

/ Michael A. Schaefer Site Vice President

Attachment: Unit 2 LER 2019-001-00, Equipment Clearance Order Error Leads to Loss of Primary Containment Integrity

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cc:

Regional Administrator, Region IV U.S. Nuclear Regulatory Commission 1600 E. Lamar Boulevard Arlington, TX 76011-4511

## Attachment

## Unit 2 LER 2019-001-00

Equipment Clearance Order Error Leads to Loss of Primary Containment Integrity

	ORM 36	6				U.S. N	UCLEAF	REGUL	ATOR	Y COM	MISSION					4 EXPIRES	
(04-2018)			LICENSEE EVENT REPORT (LER) (See Page 2 for required number of digits/characters for each (See NUREG-1022, R.3 for instruction and guidance for completin http://www.nrc.gov/reading-rm/doc-collections/nuregs/staff/sr102					ach bloc leting thi	is form	Reported industry. (T-2 F43) to Infoco Regulator Washingt display a	Estimated burden per response to comply with this mandatory collection request 80 hox Reported lessons learned are incorporated into the licensing process and fed back industry. Send comments regarding burden estimate to the Information Services Brar (T-2 F43), U.S. Nuclear Regulatory Commission, Washington, DC 2055-0001, or by e-n to Infocollects.Resource@nrc.gov, and to the Desk Officer, Office of Information Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budg Washington, DC 20503. If a means used to impose an information collection does display a currently valid OMB control number, the NRC may not conduct or sponsor, an person is not required to respond to, the information collection.			and fed back to Services Branch 0001, or by e-mail Information and ent and Budget, plection does not			
1. Faci	ility Nar	ne							2	2. Dock	et Numb						
Sout	h Tex	as Unit	2							0500	0499			1	of 5		94. 
South Texas Unit 2 05000499																	
	4. Title Equipment Clearance Order Error Leads to Loss of Primary Containment Integrity																
5.	Event I	Date	6	. LER Nu	mber		7. Report Date				8. Other Facilities Involved						
Month	Day	Year	Year	Seque Num		Rev No.	Month	Day	Yea		acility Name /A	9			Docket Ni 05000		
09	03	2019	2019	- 00	D1 –	00	11	04	20 <sup>,</sup>	40	acility Name I/A	)			Docket Ni 05000		
9. O	perating	Mode			11. Th	is Repor	t is Subn	nitted Pu	Irsuan	nt to the	Requirem	nents of	10 CFR§	: (Check	all that appi	y)	
			20.22	201(b)			0.2203(	a)(3)(i)			50.73	(a)(2)(ii)	(A)		50.73	8(a)(2)(viii)(A	)
			20.2201(d)				20.2203(a)(3)(ii)				☐ 50.73(a)(2)(ii)(B)			50.73	☐ 50.73(a)(2)(viii)(B)		
1			20.2203(a)(1)				20.2203(a)(4)				☐ 50.73(a)(2)(iii)			50.73	☐ 50.73(a)(2)(ix)(A)		
			20.2203(a)(2)(i)				50.36(c)(1)(i)(A)				☐ 50.73(a)(2)(iv)(A)			50.73	☐ 50.73(a)(2)(x)		
10.	10. Power Level		□ 20.2203(a)(2)(ii) □				☐ 50.36(c)(1)(ii)(A)				☐ 50.73(a)(2)(v)(A)			73.7	☐ 73.71(a)(4)		
			20.2203(a)(2)(iii)				50.36(c)(2)				☐ 50.73(a)(2)(v)(B)			73.7	☐ 73.71(a)(5)		
			20.2203(a)(2)(iv)				☐ 50.46(a)(3)(ii)				⊠ 50.73(a)(2)(v)(C)			73.77	′(a)(1)		
	100		20.2203(a)(2)(v)				☐ 50.73(a)(2)(i)(A)				50.73	(a)(2)(v)	(D)		73.77	7(a)(2)(ii)	
			20.2203(a)(2)(vi)			⊠ 5	⊠ 50.73(a)(2)(i)(B)				50.73	(a)(2)(vi	i)		73.7	7(a)(2)(iii)	
□ 50.73(a)(2)(i)(C) □ Other (Specify in Abstract below or in NRC Form 366A					A												
Liconoo	- Conto						12. Lice	ensee C	ontac	ct for th	nis LER			Teleph	one Number	(Include Area C	ode)
License Tim			Licensi	ng Eng	ineer									1 1	361) 97:	•	
					13. Comp						ure Descril						
C	Cause	Syste		mponent	Manufact	1	Reportable			Cau		System	Compo		Manufactur	-	table To ICES
	A	NI		PEN	N/A		Y			N/	A	N/A	N//	<u> </u>	N/A Month	Day	N/A Year
			4. Supple											N/A			
1			5. Expected			No Daced typ	ewritten li	nes)									
	or (Entites	o 1100 opu	555, i.o., apj			96		,									
co vic an co pa vic to re to	On September 5, 2019, it was discovered that an Equipment Clearance Order (ECO) to replace a degraded containment isolation valve on the outside of the Reactor Containment Building (RCB) had an error that resulted in a violation of Technical Specifications. The ECO incorrectly opened a test connection between the containment wall and a containment isolation valve inside the RCB that was required to be closed for Technical Specification compliance. When the degraded containment isolation valve on the outside of containment was removed, a leakage path out of containment was established. This resulted in a loss of one of the three fission product barriers and a violation of two Technical Specifications. Once the ECO error was recognized, Operations corrected the valve lineup to bring Unit 2 back into compliance with Technical Specifications. The root cause of the event was personnel error related. The two Licensed Senior Reactor Operators involved in the technical review and approval of the ECO failed to recognize at-risk behaviors and apply appropriate human performance tools. Corrective actions planned include procedure changes, as well as modifications to the software that controls the ECO process.																

NRC	FOF	RM 366A U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010	4	EXPIRES:	3/31/2020	
(04-2018) LICENSEE EVENT RE CONTINUATION S (See NUREG-1022, R.3 for instruction and guidance for http://www.nrc.gov/reading-rm/doc-collections/nuregs/str		LICENSEE EVENT RI CONTINUATION S	EPORT (LER) SHEET	Estimated burden per response to comply with this mandatory collection request 80 hours. Rep lessons learned are incorporated into the licensing process and fed back to industry. Send comr regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regu Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, a the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Offi Management and Budget, Washington, DC 20503. If a means used to impose an inform				
1. F			2. DOCKET NUMBER		3. LER NUMBER		1	
Sou	uth <sup>-</sup>	Texas Unit 2	05000-499		YEAR	SEQUENTIAL NUMBER	REV NO.	
				2019	- 001	- 00		
	<ul> <li>NARRATIVE</li> <li>Description of Reportable Event</li> <li>A. Reportable event classification This event is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B) as a condition prohibited by the plant's Technical Specifications. Technical Specifications 3.6.3 (Containment Isolation Valves) and 3.6.1.1 (Primary Containment) were not met because containment penetration M-82D was not isolated within 24 hours as required and primary containment integrity was not restored within 1 hour as required nor was the plant placed in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours as required. This event is reportable pursuant to 10 CFR 50.73(a)(2)(v)(C) and (D) as an event or condition that could have prevented fulfillment of the safety function of structures or systems that are needed to (C) control the release of radioactive material or (D) mitigate the consequences of an accident. Both the inside and outside isolation valves for containment penetration M-82D were inoperable in a required mode. The inoperability was due to one or more personnel errors and no redundant equipment in the same system (individual containment penetration) was operable and available to perform the required safety function of containment isolation.</li></ul>							
		Plant operating conditions prior to ev						
		Prior to the event on September 3	, 2019, Unit 2 was	operating in Mode 1 at 100%	% power.			
		Status of structures, systems, and co to the event	pmponents that were	e INOPERABLE at the start o	f the event	and that contrib	uted	
		At the start of the event, Containm [ISV] was inoperable due to exces			Isolation \	/alve 2-AP-FV-:	2456	
	D.	Background information						

Primary containment integrity ensures that the release of radioactive materials from the containment atmosphere will be restricted to those leakage paths and associated leak rates assumed in the safety analyses. This limits site boundary radiation doses to within federal dose guideline values during accident conditions. Operability of containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment. During MODES 1, 2, 3, and 4 without primary containment integrity, Technical Specification 3.6.1.1 requires restoration of containment integrity within 1 hour or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours. During MODES 1, 2, 3, and 4 with one or more containment isolation valve(s) inoperable, Technical Specification 3.6.3 requires that at least one isolation barrier be maintained operable in each affected penetration.

## E. Narrative summary of the event

Timeline (Note: All times are Central Standard Time):

July 10, 2019 [1515]: Containment Air Sample Outside Reactor Containment Isolation Valve 2-AP-FV-2456 declared inoperable due to excessive seat leakage.

		RM 366A U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010	4	EXPIRES: 3	3/31/2020
) (Se		LICENSEE EVENT RI CONTINUATION S IREG-1022, R.3 for instruction and guidance for	SHEET	Estimated burden per response to comply with this mandatory collection request: 80 hours. Report lessons learned are incorporated into the licensing process and fed back to industry. Send commer regarding burden estimate to the Information Services Branch (T-2 F43), U.S. Nuclear Regulate Commission, Washington, DC 20555-0001, or by e-mail to Infocollects.Resource@nrc.gov, and the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office Management and Budget, Washington, DC 20503. If a means used to impose an informatii collection does not display a currently valid OMB control number, the NRC may not conduct sponsor, and a person is not required to respond to, the information collection.			
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					YEAR	SEQUENTIAL	REV
S	outh	Texas Unit 2	05000-499		2019	NUMBER - 001	νο. - 00
NA	NARRATIVE						
	E.	Narrative summary of the event (con	tinued)				
		August 1, 2019: ECO revision prep	pared for replacem	ent of Containment Isolatior	n Valve 2-/	\P-FV-2456.	
		August 26, 2019: ECO revision ap	proved by the Tecl	nnical Reviewer (Licensed S	Senior Rea	ctor Operator).	
		September 2, 2019 [1836]: ECO re Operator). The Technical Reviewe				or Reactor	
	September 3, 2019 [1300 - 1315]: Event Date. Containment Hydrogen Monitoring Sample Inlet Test Valve 2-CM-0005 [TV] between the containment wall and a containment isolation valve inside the containment building is opened as directed by the ECO. Opening of this valve coupled with the inoperability of Containment Isolation Valve 2-AP-FV-2456 created a breach of containment.						
		September 5, 2019 [2115]: Discov and 3.6.1.1 associated with contai				ifications 3.6.3	
		September 5, 2019 [2322]: Valve 2	2-CM-0005 closed	and Technical Specification	complian	ce restored.	
	F.	Method of discovery					
		The event was discovered by a Ur	nit Supervisor appr	oving a release revision (wo	rk comple	tion) of the ECC	).
11.	Co	mponent failures					
	A.	Failure Mode, mechanism, and effec	ts of failed compone	ent			
		The failed component in this even containment integrity) of containm containment isolation valve open.	t is containment pe ent penetration M-i	netration M-82D [PEN]. The 82D could not be met with b	e safety fu oth an ins	nction (maintain ide and outside	
	В.	Cause of component failure	,				
		Containment penetration M-82D failure was due to an error in an ECO which went undetected by two utility licensed SROs as a result of their failure to recognize at-risk behaviors and apply appropriate human performance tools.					
	C.	Systems or secondary functions that were affected by failure of components with multiple functions					
		No additional systems were affect	ed by the containm	ent penetration failure.			
	D.	Failed component information					
		Reactor Containment Building [NH] Penetration [PEN]					

	FORM 366A U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010	4	EXPIRES: 3	3/31/2020	
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	//www.nrc.gov/reading-rm/doc-collections/nuregs/s ACILITY NAME	2. DOCKET N	sponsor, and a person is not required to respond to, the information collection. UMBER 3. LER NUMBER				
		2. DOCKET N	JWDER	YEAR	SEQUENTIAL	REV	
Sou	ith Texas Unit 2	05000-499			NUMBER	NO.	
				2019	- 001	- 00	
NAR	RATIVE						
III. A	Analysis of the event						
<i> </i>	A. Safety system responses that occurr	ed					
	No safety systems were required to	o respond as a res	ult of this event.				
E	3. Duration of safety system inoperabilit	У					
	The duration of the containment br September 5, 2019 (2322)].	each was approxir	nately 58 hours [September	3, 2019 (	1300) to		
	C. Safety consequences and implication	าร					
	This event had insignificant safety consequences because the containment breach was disconnected from the Reactor Coolant System (RCS) by a series of closed valves for the duration of the event. Additionally, the lines to the vent and test connection valves which were found to be open, as well as the main line connecting them and passing through 2-FV-2456, all have a one-inch inner diameter. Containment breaches of less than a three-inch diameter do not lead to a large radiation release. The event did not result in any offsite release of radioactivity or increase of offsite dose rates, and there were no personnel injuries or damage to any other safety-related equipment associated with this event.						
	Therefore, there was no adverse e	ffect on the health	and safety of the public.				
IV. C	Cause of the event						
f	The root cause of the event was personnel error related. Individuals involved in the technical review and approval of the ECO failed to recognize at-risk behaviors and apply appropriate Human Performance (HU) tools. At-risk behaviors not considered include:						
E	Making assumptions - The Issuing Autho ECO revision and did not perform a detai wrong assumption that an inside containr	led review of the rer	naining ECO line items. As a				
	Believing the source of information is abs ECOs would be technically sound and th					that	
Thinking the task is routine or simple - Both the Technical Reviewer and Approver of the ECO stated they viewer routine or simple.							
ļ	Appropriate error reduction tools not used	d by both individuals	include Questioning Attitude	and Peer F	Review/Collabora	ation.	
V. C	Corrective actions						
	Completed - Closed valve 2-CM-0005 to (disciplinary) process for responsible pers		ent integrity and implemente	d the site c	consistency matri	ix	
	Planned - Procedure changes and modification to the software that controls the ECO process. The procedure changes will:						

NRC FORM 366A U.S. NUCLEAR REGULA	TORY COMMISSION	APPROVED BY OMB: NO. 3150-010	4	EXPIRES:	3/31/2020
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1. FACILITY NAME	2. DOCKET N			3, LER NUMBER	
South Texas Unit 2	05000-499		YEAR	SEQUENTIAL NUMBER	REV. NO.
			2019	- 001	- 00
NARRATIVE					
<ul> <li>V. Corrective actions (continued)</li> <li>(1) Require that the Technical Reviewer a that ensure Technical Specification comp</li> <li>(2) Add guidance to require SROs that us Assessments to sign on to the ECO as an</li> <li>(3) Include steps and acceptor checklist it and continuing Technical Specification co the Issuing Authority and their sole function direction to ensure the ECO is designated The software modifications will implement</li> </ul>	liance. Se ECOs to ensure T In Acceptor specification tems for the ECO join ompliance. The ECC on / purpose is to en d as being used for T	Fechnical Specification compli Ily for Technical Specification b acceptor function for Operat acceptor SHALL be a separa sure Technical Specification compli	ance on Op compliance ions use wh ate SRO fro compliance. ance.	perability a. men ensuring init m the SRO who Also include	tial o is
separate SRO from the SRO who is the live of the separate SRO from the SRO who is the live of the separate sepa	ssuing Authority for	ECOs used to maintain Techr	nical Specifi	ication compliar	ice.
An operating experience review identified (LER) 2000-003-00, Reactor Containmer meet management expectations associat for maintaining the unit in compliance with the Unit Supervisor verified the ECO to en Authority's capability. The fact that the va and Unit Supervisors to perceive this acti- responsible for ensuring the use of peer of were performed. Corrective actions include regarding roles and responsibilities for Te- tools.	nt Building Penetration ted with work praction n Technical Specific nsure Technical Specific nsure Technical Specific live was already tagg vity to be a low-risk checks for Technica ded Operations mar	on M-85 Not Properly Isolated ces. The Shift and Unit Superv ations. In the LER 2000-003-0 ecification compliance due to o ged to comply with Technical evolution. In addition, the Shift I Specification compliance and nagement reinforcing expectat	, was attribu- risors are ul 00 event, ne overconfide Specificatio and Unit S I no peer ch ions to Ope	uted to failure to timately respon- either the Shift n nce in the Work ns caused the S supervisors are necks for the EC erations personr	sible or Start Shift CO iel
The LER 2000-003-00 event is very simil performance errors. Each event involves Specifications. The ECO review inadequa performance tools, including peer checkir based solutions (i.e., management reinfo focused on both behavior-based solutions and software changes).	inadequate reviews acies in each event ng. LER 2000-003-0 rcing expectations to	of an ECO resulting in non-co are attributed to not using prop 0 corrective actions were excl o subordinates), whereas corr	ompliance v per error rec usively focu ective action	vith Technical duction or huma used on behavio ns for this event	or- t are