

JAN 13 1978

Docket Nos. 50-269
50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam Production
Post Office Box 2178
422 South Church Street
Charlotte, North Carolina 28242

Gentlemen:

RE: OCOFEE NUCLEAR STATION, UNITS 1, 2 & 3

By letter dated December 17, 1976, we sent you a document entitled "NRC Staff Guidance for Complying with Certain Provisions of 10 CFR 50.55a(g), Inservice Inspection Requirements". In addition to clarifying the proper methods for complying with the regulation, this guidance provided a general outline of the type of information that the NRC staff would need to review inservice inspection and testing programs, and to evaluate requests for relief from ASME Code requirements that are determined to be impractical for a facility.

After reviewing a number of submittals relating to s50.55a(g) requirements from various licensees, we have concluded that additional guidance would be useful to all licensees to aid in the preparation of these submittals, and to expedite the NRC staff review and approval of the proposed programs and any requests for relief from certain ASME Code requirements. The need for this guidance is particularly evident for the pump and valve testing requirements.

Enclosed for your use is the "NRC Staff Guidance for Preparing Pump and Valve Testing Program Descriptions and Associated Relief Requests Pursuant to 10 CFR 50.55a(g)". This enclosure defines the required scope of a pump and valve testing program, itemizes the specific information needed for staff review, and provides guidelines for submitting information to support requests for relief from any ASME Code requirements found to be impractical for a facility. The same information is being sent to all nuclear power plant licensees and is intended to complement and expand on the guidance we provided to

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you in our previous letter. Although the enclosure specifically addresses pump and valve testing requirements only, the same level of detailed information identified in this guidance should also be provided in inservice inspection program submittals.

We request that you follow the enclosed guidance to the greatest extent possible when submitting proposed inservice inspection and testing programs and requests for relief from ASME Code requirements, and when responding to additional information requests from the staff. Your adherence to this guidance will minimize the NRC staff review time needed to approve your proposed programs and associated relief requests.

If you have any questions regarding implementation of 10 CFR 50.55a(g) at your facility, please contact us.

Sincerely,

T. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

g/a

Enclosures:
NRC Staff Guidance

cc w/enclosure:
See next page

- DISTRIBUTION
- Dockets
- NRC PDRs
- LOCAL PDR
- ORB#1 Reading
- ASchwencer
- SMSheppard
- DNeighbors
- OELD
- JWetmore
- I&E(3)
- DEisenhut
- TBAbernathy
- JRBuchanan
- ACRS(16)

POOR ORIGINAL

OFFICE →	DOR:ORB#1	DOR:5	DOR:ORB#1		
BURNAME →	DNeighbors:1b	JWetmore	ASchwencer		
DATE →	1/12/78	1/12/78	1/13/78		

cc: Mr. William L. Porter
Duke Power Company
P. O. Box 2178
422 South Church Street
Charlotte, North Carolina 28242

J. Michael McGarry, III, Esquire
DeBevoise & Liberman
700 Shoreham Building
806-15th Street, NW.,
Washington, D.C. 20005

Oconee Public Library
201 South Spring Street
Walhalla, South Carolina 29691



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 13, 1978

Docket Nos. 50-269
50-270
and 50-287

Duke Power Company
ATTN: Mr. William O. Parker, Jr.
Vice President - Steam Production
Post Office Box 2175
422 South Church Street
Charlotte, North Carolina 28202

Gentlemen:

RE: DOONEE NUCLEAR STATION, UNITS 1, 2 & 3

By letter dated December 17, 1976, we sent you a document entitled "NRC Staff Guidance for Compliance with Certain Provisions of 10 CFR 50.55a(-), Inservice Inspection Requirements". In addition to clarifying the proper methods for complying with the regulation, this guidance provided a general outline of the type of information that the NRC staff would need to review inservice inspection and testing programs, and to evaluate requests for relief from ASME Code requirements that are determined to be impractical for a facility.

After reviewing a number of submittals relating to s50.55a(g) requirements from various licensees, we have concluded that additional guidance would be useful to all licensees to aid in the preparation of these submittals, and to expedite the NRC staff review and approval of the proposed programs and any requests for relief from certain ASME Code requirements. The need for this guidance is particularly evident for the pump and valve testing requirements.

Enclosed for your use is the "NRC Staff Guidance for Preparing Pump and Valve Testing Program Descriptions and Associated Relief Requests Pursuant to 10 CFR 50.55a(g)". This enclosure defines the required scope of a pump and valve testing program, itemizes the specific information needed for staff review, and provides guidelines for submitting information to support requests for relief from any ASME Code requirements found to be impractical for a facility. The same information is being sent to all nuclear power plant licensees and is intended to complement and expand on the guidance we provided to

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We request that you follow the enclosed guidance to the greatest extent possible when submitting proposed inservice inspection and testing programs and requests for relief from ASME Code requirements, and when responding to additional information requests from the staff. Your adherence to this guidance will minimize the NRC staff review time needed to approve your proposed programs and associated relief requests.

If you have any questions regarding implementation of 10 CFR 50.55a(g) at your facility, please contact us.

Sincerely,

J. V. Wenzel

for
A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosure:
NRC Staff Guidance

cc w/enclosure:
See next page