



Public Service Company of Colorado

16805 Weld County Road 19 1/2, Platteville, Colorado 80651

April 9, 1980
Fort St. Vrain
Unit No. 1
P-80075

Mr. Karl V. Seyfrit, Director
Nuclear Regulatory Commission
Region IV
Office of Inspection and Enforcement
611 Ryan Plaza Drive
Suite 1000
Arlington, Texas 76012

Reference: Facility Operating License
No. DPR-34

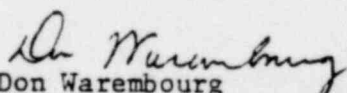
Docket No. 50-267

Dear Mr. Seyfrit:

Enclosed please find a copy of Reportable Occurrence Report No. 50-267/80-13, Final, submitted per the requirements of Technical Specification AC 7.5.2(b)2.

Also, please find enclosed one copy of the Licensee Event Report for Reportable Occurrence Report No. 50-267/80-13.

Very truly yours,


Don Warembourg
Manager, Nuclear Production

DW/cls

Enclosure

cc: Director, MIPC

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REPORT DATE: April 9, 1980

REPORTABLE OCCURRENCE 80-13

OCCURRENCE DATE: March 10, 1980

ISSUE 0

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FORT ST. VRAIN NUCLEAR GENERATING STATION
PUBLIC SERVICE COMPANY OF COLORADO
16805 WELD COUNTY ROAD 19 1/2
PLATTEVILLE, COLORADO 80651

REPORT NO. 50-267/80-13/03-L-0

Final

IDENTIFICATION OF
OCCURRENCE:

On March 10, 1980, the indicated sample flow for two of the primary coolant dewpoint moisture monitors was discovered to be less than required by LCO 4.4.1, Note (t).

This is reportable per Fort St. Vrain Technical Specification AC 7.5.2(b)2.

EVENT
DESCRIPTION:

On March 10, 1980, while operating at 58% thermal power and 182 MW electrical, it was discovered that the indicated sample flow for two of the low level primary coolant dewpoint moisture monitors was less than required by LCO 4.4.1, Note (t).

Instrument personnel investigated the sample flow problems and found that the flow elements for MM-1117 and MM-1122 were reading incorrectly. See Figure 1. The indicated flow through the flow element ①, was less than required for the moisture monitor to be considered operable. However, considering system conditions and the position of the bypass valve ②, the actual flow should have been greater than the indicated and the required flow. Plant power level was reduced, and as the actual sample flow rate decreased, the sample flow indicated by the flow element ① began to correspond to the expected actual flow rate through the moisture monitor ③.

The Instrument personnel could not determine what caused the erroneous readings in either of the flow elements, and it was decided to replace both of the flow elements.

CAUSE
DESCRIPTION:

The insufficient indicated sample flow was the result of incorrect readings from respective moisture monitor flow elements.

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CORRECTIVE
ACTION:

The flow element for MM-1122 was replaced and the flow element for MM-1117 will be replaced as plant conditions allow.

There was no affect on public health or safety.

No further corrective action is anticipated or required.

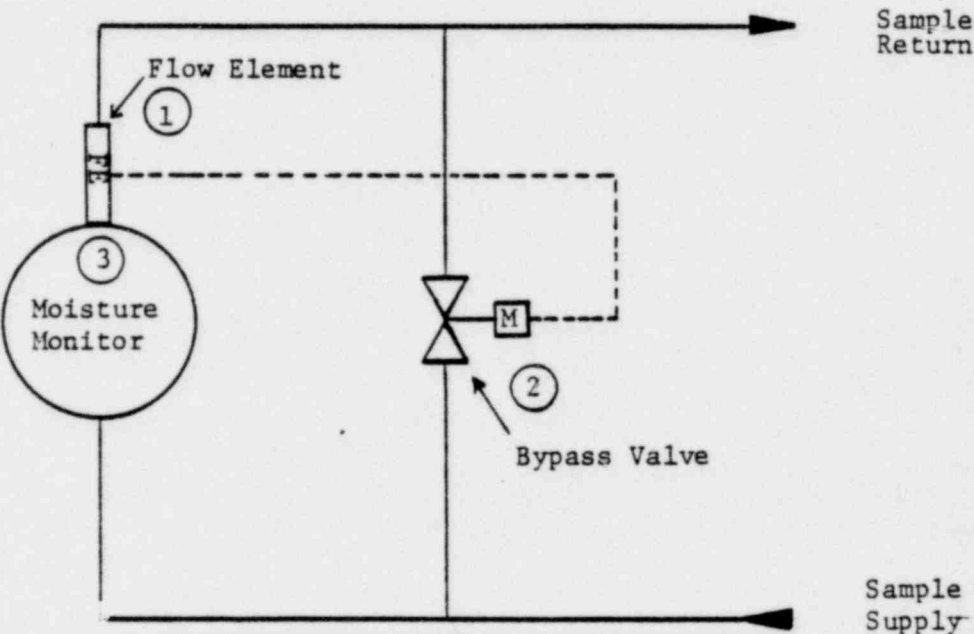


FIGURE 1

Prepared By: Asa B. Reed
Asa B. Reed
Technical Services Technician

Reviewed By: J. W. Gahm
J. W. Gahm
Technical Services Supervisor

Reviewed By: Frank M. Mathie
Frank M. Mathie
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Don Warembourg
Manager, Nuclear Production