



Department of Energy  
 Clinch River Breeder Reactor  
 Plant Project Office  
 P.O. Box U  
 Oak Ridge, Tennessee 37830  
 Docket No. 50-537

April 11, 1980

Mr. Denwood F. Ross  
 Acting Director  
 Division of Project Management  
 Office of Nuclear Reactor Regulation  
 U.S. Nuclear Regulatory Commission  
 Washington, D.C. 20555

Dear Mr. Ross:

TOPICAL REPORT ON CRBRP SCALE MODEL RESPONSE TO A SIMULATED HCDA

Enclosed are seventy (70) copies of Revision 1 to WARD-D-0218, "Structural Response of CRBRP Scale Models to a Simulated Hypothetical Core Disruptive Accident," dated October 1979. In this revision eight changes are made. The most important change provides a corrected "pre-test" prediction of the strain profile for one of the test vessels (SM-2). In addition, some data errors are corrected in one table and the remaining six changes deal with poor reproduction and minor editing errors.

None of these changes alter the conclusion of the tests which show that the CRBRP reactor vessel and head can withstand the loads resulting from a postulated HCDA, that application of the computer codes can be pursued with an increased confidence, and confirms the conservatism of the methods used to generate loadings from the postulated event.

Sincerely,

Raymond L. Copeland  
 Acting Assistant Director  
 for Public Safety

PS:80:116

Enclosure: Topical Report (70)

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