



Idaho National Engineering Laboratory

April 29, 1986

Mr. Conrad McCracken
Section Leader, PWR Directorate #6
Division of PWR Licensing B
Bethesda, MD 20014

TRANSMITTAL OF UPDATED STATUS OF DAVIS-BESSE CONFIRMATORY ITEMS LISTED IN SECTION V OF INEL REPORT, TECHNICAL EVALUATION OF DAVIS-BESSE NUCLEAR POWER STATION SYSTEM REVIEW AND TEST PROGRAM (SRTP) EGG-NTA-7104, - Oben-46-86

Ref: G. E. Marx ltr to S. B. Milam, Marx-96-86, Transmittal of the NRC Form 189 Revision 2 For Selected Operating Plant Licensee Issues Program (FIN D6022), March 19, 1986


Dear Mr. McCracken:

In accordance with the Referenced 189, INEL issued a Technical Evaluation Report (TER) on the Davis-Besse SRTP in January, 1986. This TER documented 10 items which the INEL review determined to be confirmatory items, requiring additional review.

The purpose of this memo is to provide an updated status of these items, for your information and possible inclusion into your Safety Evaluation Report.

The January TER documented the following items as confirmatory.

1. The gaseous radwaste system oxygen monitors which are relied upon to prevent a postulated hydrogen explosion will be addressed in the "supervisory review" process.
2. The post accident sampling system and the recognition of the importance of the system to evaluate post accident conditions will be addressed in the "supervisory review" process.
3. The utility has committed to perform a supervisory review of the fuel handling area ventilation system to determine its applicability to "systems important to safe plant operation" following restart. This review should be verified complete and acceptable prior to moving fuel.

 **EG&G** Idaho, Inc. P.O. Box 1625 Idaho Falls, ID 83415

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4. For the miscellaneous containment isolation valves not included in the scope of the 31 systems, Toledo Edison will perform a "supervisory review of these valves consistent with the review conducted on the isolation valves of the 31 systems. NRC/Contractor review of the above listed "supervisory reviews" is considered to be a confirmatory item.
5. The Make-up and Purification System and system functions identified in the IPRC approved SRTP dated December 3, 1985 did not specifically address the function of Letdown Isolation on a Hi Temperature signal to mitigate a letdown line pipe break. The utility committed to revise the Davis Besse USAR to clarify what systems and system functions are assumed to mitigate a letdown line pipe break discussed in USAR Section 15.4. This action remains to be confirmed.
6. The Anticipatory Reactor Trip System and System functions identified in the IPRC approved SRTP dated December 2, 1985 did not specifically address the Interlock Function that prevents reset before the initiating signal has cleared. The utility committed to verify proper operation of this interlock during the system testing. This action remains to be confirmed.
7. The Control Rod Drive System and system functions identified in the IPRC approved SRTP dated December 2, 1985 did not specifically address the Reactor Runback to 15% power. Runbacks are addressed in the Integrated Control System and system functions identified in the IPRC approved SRTP dated November 14, 1985. However the ICS function does not address the runback function as a safety function. The Main Steam System and system functions identified in the IPRC approved SRTP dated December 3, 1985 did not specifically address the function of the turbine bypass. The utility committed to revise the USAR to update the discussion provided in Section 15.2 on the turbine trip analysis to indicate that the anticipatory reactor trip system provides the mitigating functions and delete the discussion concerning reactor runback and turbine bypass as mitigating functions. This action remains to be confirmed.
8. The Control Room Normal and Emergency Ventilation System identified in the IPTC approved SRRP dated December 3, 1985 did not specifically address the function of control room isolation on Auxiliary Building Hi Radiation. The utility agreed this function should be included and committed to add this function to the current list. This action remains to be confirmed.
9. Since the SRTP is continuing to develop as the various phases of program activity are initiated, performed, and reviewed, such things as the appropriate SRTP reliance on integrated plant tests to which Toledo Edison has committed, remains to be confirmed.

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10 The review of selected test outlines to ensure that they encompass all system functions required for safe plant operation and that the systems are tested under anticipated operating conditions still requires evaluation. In addition, the witnessing and evaluating of selected systems tests to determine their adequacy remains an open item due to the lack of approved testing procedures and the lack of testing to date.

As a result of the on-going INEL review of the SRTP and on-site test witnessing, the following items should be considered confirmed and closed:

Items 1, 2, 3, 4, 6, and 8.

Items 5, 7, 9, and 10 still require additional evaluation and/or utility action before they should be considered confirmed and closed.

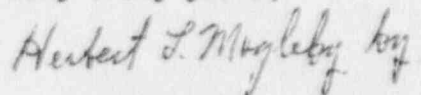
Items 5 and 7 require utility action to revise the Davis Besse USAR.

Items 9 and 10 require utility action and additional NRC and/or contractor evaluation. Although the SRTP has progressed significantly since the INEL TER was issued, reliance on integrated plant testing documented in Item 9, has not been fully defined and the test outlines and performance of testing has not yet progressed to the point of being able to confirm these items.

INEL will continue to make plant visits and follow these items as requested by you or Chris Vandenburg of NRC Region III. We will inform you, at the earliest possible date of any changes to the status of these items.

If you have any questions on this transmittal or if we can be of further assistance at this time, please contact Stan Bruske at 208-526-9345 or (FTS) 583-9345.

Very truly yours,



C. F. Obenchain, Manager
NRR and I&E Support

SB:acf

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