ATTACHMENT TO LICENSE AMENDMENT NO. 166

TO FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Facility Combined License No. NPF-92 with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Facility Combined License No. NPF-92

REMOVE	INSERT
7	7

Appendix A to Facility Combined License Nos. NPF-91 and NPF-92

REMOVE	INSERT
1.1-1	1.1-1
3.1.8-2	3.1.8-2
3.2.3-1	3.2.3-1
3.3.1-2	3.3.1-2
3.3.1-3	3.3.1-3
3.3.1-4	3.3.1-4
3.3.1-5	---
3.3.1-6	3.3.1-5
3.3.1-7	3.3.1-6
3.3.2-3	3.3.2-3
3.3.3-3	3.3.3-3
3.3.4-2	3.3.4-2
3.3.4-3	3.3.4-3
3.3.6-2	3.3.6-2
3.3.8-6	3.3.8-6
3.3.10-4	3.3.10-4
3.3.11-2	3.3.11-2
3.3.13-3	3.3.13-3
3.3.14-3	3.3.14-3
3.3.15-2	3.3.15-2

continued on next page

Appendix A to Facility Combined License Nos. NPF-91 and NPF-92 (cont'd)

REMOVE	INSERT
3.3.16-3	3.3.16-3
3.3.16-4	3.3.16-4
3.3.17-2	3.3.17-2
3.3.17-3	3.3.17-3
3.3.19-1	3.3.19-1
3.3.20-2	3.3.20-2
3.3.20-3	3.3.20-3
5.5-13	5.5-13

(7) Reporting Requirements

(a) Within 30 days of a change to the initial test program described in UFSAR Section 14, Initial Test Program, made in accordance with 10 CFR 50.59 or in accordance with 10 CFR Part 52, Appendix D, Section VIII, "Processes for Changes and Departures," SNC shall report the change to the Director of NRO, or the Director's designee, in accordance with 10 CFR 50.59(d).

(b) SNC shall report any violation of a requirement in Section 2.D.(3), Section 2.D.(4), Section 2.D.(5), and Section 2.D.(6) of this license within 24 hours. Initial notification shall be made to the NRC Operations Center in accordance with 10 CFR 50.72, with written follow up in accordance with 10 CFR 50.73.

(8) Incorporation

The Technical Specifications, Environmental Protection Plan, and ITAAC in Appendices A, B, and C, respectively of this license, as revised through Amendment No. 166, are hereby incorporated into this license.

(9) Technical Specifications

The technical specifications in Appendix A to this license become effective upon a Commission finding that the acceptance criteria in this license (ITAAC) are met in accordance with 10 CFR 52.103(g).

(10) Operational Program Implementation

SNC shall implement the programs or portions of programs identified below, on or before the date SNC achieves the following milestones:

(a) Environmental Qualification Program implemented before initial fuel load;

(b) Reactor Vessel Material Surveillance Program implemented before initial criticality;

(c) Preservice Testing Program implemented before initial fuel load;

(d) Containment Leakage Rate Testing Program implemented before initial fuel load;

(e) Fire Protection Program

1. The fire protection measures in accordance with Regulatory Guide (RG) 1.189 for designated storage building areas (including adjacent fire areas that could affect the storage area) implemented before initial receipt

1.0 USE AND APPLICATION

1.1 Definitions

- NOTE -

The defined terms of this section appear in capitalized type and are applicable throughout these Technical Specifications and Bases.

Term	Definition
ACTIONS	ACTIONS shall be that part of a Specification that prescribes Required Actions to be taken under designated Conditions within specified Completion Times.
ACTUATION LOGIC TEST	An ACTUATION LOGIC TEST shall be the application of various simulated or actual input combinations in conjunction with each possible interlock logic state required for OPERABILITY of a logic circuit and the verification of the required logic output. The ACTUATION LOGIC TEST may be performed by means of any series of sequential, overlapping, or total steps.
AXIAL FLUX DIFFERENCE (AFD)	AFD shall be the difference in normalized flux signals between the top and bottom halves of a two-section excore neutron detector.
CHANNEL CALIBRATION	A CHANNEL CALIBRATION shall be the adjustment, as necessary, of the channel output such that it responds within the necessary range and accuracy to known values of the parameter that the channel monitors. The CHANNEL CALIBRATION shall encompass all devices in the channel required for OPERABILITY. Calibration of instrument channels with resistance temperature detector (RTD), thermocouple, or reactor coolant pump speed sensors may consist of an inplace qualitative assessment of sensor behavior and normal calibration of the remaining adjustable devices in the channel. The CHANNEL CALIBRATION may be performed by means of any series of sequential, overlapping, or total channel steps.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and Associated Completion Time of Condition C not met.	D.1 Be in MODE 3.	15 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.1.8.1	Verify the RCS lowest loop average temperature is ≥ 541°F.	30 minutes
SR 3.1.8.2	Verify THERMAL POWER is ≤ 5% RTP.	30 minutes
SR 3.1.8.3	Verify SDM is within the limits specified in the COLR.	24 hours

3.2 POWER DISTRIBUTION LIMITS

3.2.3 AXIAL FLUX DIFFERENCE (AFD) (Constant Axial Offset Control (CAOC) Methodology)

LCO 3.2.3 The AFD:

a. Shall be maintained within the target band specified in the COLR about the target flux difference.

b. May deviate outside the target band with THERMAL POWER < 90% RTP, but ≥ 50% RTP, provided AFD is within the acceptable operation limits specified in the COLR and cumulative penalty deviation time is ≤ 1 hour during the previous 24 hours.

c. May deviate outside the target band with THERMAL POWER < 50% RTP.

- NOTES -

1. The AFD shall be considered outside the target band when two or more OPERABLE excore channels indicate AFD to be outside the target band.

2. With THERMAL POWER ≥ 50% RTP, penalty deviation time shall be accumulated on the basis of a 1 minute penalty deviation for each 1 minute of power operation with AFD outside the target band.

3. With THERMAL POWER < 50% RTP and > 15% RTP, penalty deviation time shall be accumulated on the basis of a 0.5 minute penalty deviation for each 1 minute of power operation with AFD outside the target band.

4. A total of 16 hours of operation may be accumulated with AFD outside the target band without penalty deviation time during surveillance of Power Range Neutron Flux channels in accordance with SR 3.3.1.4, provided AFD is maintained within acceptable operation limits.

APPLICABILITY: MODE 1 with THERMAL POWER > 15% RTP and with the On-Line Power Distribution Monitoring System (OPDMS) not monitoring parameters.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
E. As required by Required Action C.1 and referenced in Table 3.3.1-1.	E.1 Reduce THERMAL POWER to < P-10.	6 hours

SURVEILLANCE REQUIREMENTS

- NOTE -

Refer to Table 3.3.1-1 to determine which SRs apply for each RTS Function.

SURVEILLANCE		FREQUENCY
SR 3.3.1.1	--- **- NOTES -** 1. Required to be met within 12 hours after reaching 15% RTP. 2. If the calorimetric heat balance is ≥ 15% RTP, and if the nuclear instrumentation channel indicated power is: a. lower than the calorimetric measurement by > 5% RTP, then adjust the nuclear instrumentation channel upward to match the calorimetric measurement. b. higher than the calorimetric measurement, then no adjustment is required. --- Compare results of calorimetric heat balance to nuclear instrument channel output.	24 hours

SURVEILLANCE REQUIREMENTS (continued)

	SURVEILLANCE	FREQUENCY
SR 3.3.1.2	---------- **- NOTES -** 1. Adjust the conversion factor, ΔT°, in the ΔT power calculation ($q_{\Delta T}$) if absolute difference between $q_{\Delta T}$ and the calorimetric measurement is > 3% RTP. 2. Required to be met within 12 hours after reaching 50% RTP. 3. If the calorimetric heat balance is < 70% RTP, and if $q_{\Delta T}$ is: a. lower than the calorimetric measurement by > 5%, then adjust ΔT° to match the calorimetric measurement. b. higher than the calorimetric measurement, then no adjustment is required. ----------	
	Compare results of calorimetric heat balance to the ΔT power calculation ($q_{\Delta T}$) output.	24 hours
SR 3.3.1.3	---------- **- NOTES -** 1. Adjust nuclear instrument channel in PMS if absolute difference is ≥ 1.5% AFD. 2. Required to be met within 24 hours after reaching 20% RTP. ----------	
	Compare results of the incore detector measurements to nuclear instrument channel AXIAL FLUX DIFFERENCE.	31 effective full power days (EFPD)
SR 3.3.1.4	---------- **- NOTE -** Required to be met within 24 hours after reaching 50% RTP. ----------	
	Calibrate excore channels to agree with incore detector measurements.	92 EFPD

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.1.5	---------- **- NOTE -** This Surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.1.6	---------- **- NOTE -** Neutron detectors are excluded from CHANNEL CALIBRATION. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.1.7	---------- **- NOTE -** Verification of setpoint is not required. ---------- Perform TADOT.	24 months
SR 3.3.1.8	---------- **- NOTE -** Neutron detectors are excluded from response time testing. ---------- Verify RTS RESPONSE TIME is within limits.	24 months on a STAGGERED TEST BASIS

Table 3.3.1-1 (page 1 of 2)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS
1. Power Range Neutron Flux				
a. High Setpoint	1,2	4	D	SR 3.3.1.1 SR 3.3.1.6 SR 3.3.1.8
b. Low Setpoint	1[(a)],2	4	D	SR 3.3.1.6 SR 3.3.1.8
2. Power Range Neutron Flux High Positive Rate	1,2	4	D	SR 3.3.1.6 SR 3.3.1.8
3. Overtemperature ΔT	1,2	4 (2/loop)	D	SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.8
4. Overpower ΔT	1,2	4 (2/loop)	D	SR 3.3.1.2 SR 3.3.1.3 SR 3.3.1.4 SR 3.3.1.5 SR 3.3.1.8
5. Pressurizer Pressure				
a. Low 2 Setpoint	1[(b)]	4	E	SR 3.3.1.5 SR 3.3.1.8
b. High 2 Setpoint	1,2	4	D	SR 3.3.1.5 SR 3.3.1.8
6. Pressurizer Water Level – High 3	1[(b)]	4	E	SR 3.3.1.5 SR 3.3.1.8

(a) Below the P-10 (Power Range Neutron Flux) interlocks.

(b) Above the P-10 (Power Range Neutron Flux) interlock.

Table 3.3.1-1 (page 2 of 2)
Reactor Trip System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	CONDITIONS	SURVEILLANCE REQUIREMENTS
7. Reactor Coolant Flow – Low 2	1[(b)]	4 per hot leg	E	SR 3.3.1.2 SR 3.3.1.5 SR 3.3.1.8
8. Reactor Coolant Pump (RCP) Bearing Water Temperature – High 2	1,2	4 per RCP	D	SR 3.3.1.5 SR 3.3.1.8
9. RCP Speed – Low 2	1[(b)]	4 (1/pump)	E	SR 3.3.1.5 SR 3.3.1.8
10. Steam Generator (SG) Narrow Range Water Level – Low 2	1,2	4 per SG	D	SR 3.3.1.5 SR 3.3.1.8
11. Steam Generator (SG) Narrow Range Water Level – High 3	1,2[(c)]	4 per SG	D	SR 3.3.1.5 SR 3.3.1.8
12. Passive Residual Heat Removal Actuation	1,2	4 per valve	D	SR 3.3.1.7 SR 3.3.1.8

(b) Above the P-10 (Power Range Neutron Flux) interlock.

(c) Above the P-11 (Pressurizer Pressure) interlock.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.2.2	---------- **- NOTE -** Neutron detectors are excluded from CHANNEL CALIBRATION. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.2.3	---------- **- NOTE -** Neutron detectors are excluded from response time testing. ---------- Verify RTS RESPONSE TIME is within limits.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.3.1	---------- **- NOTE -** Not required to be met in MODE 1. ---------- Perform CHANNEL CHECK.	12 hours
SR 3.3.3.2	---------- **- NOTE -** Neutron detectors are excluded from CHANNEL CALIBRATION. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.3.3	---------- **- NOTE -** Neutron detectors are excluded from response time testing. ---------- Verify RTS RESPONSE TIME is within limits.	24 months on a STAGGERED TEST BASIS

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met. OR One or more Functions with three or more channels inoperable in MODE 3, 4, or 5.	D.1 Initiate action to fully insert all rods. AND D.2 Place the Plant Control System in a condition incapable of rod withdrawal.	1 hour 1 hour

SURVEILLANCE REQUIREMENTS

- NOTE -

Refer to Table 3.3.4-1 to determine to which RTS ESFAS Function the SR applies.

SURVEILLANCE		FREQUENCY
SR 3.3.4.1	Verify RTS RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

Table 3.3.4-1 (page 1 of 1)
Reactor Trip System Engineered Safety Feature Actuation System Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS	SURVEILLANCE REQUIREMENTS
1. Safeguards Actuation Input from Engineered Safety Feature Actuation System – Automatic	1,2	4	SR 3.3.4.1
2. ADS Stages 1, 2, and 3 Actuation Input from Engineered Safety Feature Actuation System – Automatic	1,2,3[(a)],4[(a)],5[(a)]	4	None
3. Core Makeup Tank Actuation Input from Engineered Safety Feature Actuation System – Automatic	1,2,3[(a)],4[(a)],5[(a)]	4	None

(a) With Plant Control System capable of rod withdrawal or one or more rods not fully inserted.

SURVEILLANCE REQUIREMENTS

None

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.8.1	---------- **- NOTE -** Only required to be met for Source Range Neutron Flux Doubling. ---------- Perform CHANNEL CHECK.	12 hours
SR 3.3.8.2	---------- **- NOTE -** This surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.8.3	---------- **- NOTE -** Not applicable to Function 1.a. ---------- Verify ESF RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.10.1	---------- **- NOTE -** This surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.10.2	Verify ESF RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.11.1	---------- **- NOTE -** This surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.11.2	Verify ESF RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.13.1	---------- **- NOTE -** This surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.13.2	Verify ESF RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.14.1	---------- **- NOTE -** This surveillance shall include verification that the time constants are adjusted to within limits. ---------- Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.14.2	Verify ESF RESPONSE TIME is within limit.	24 months on a STAGGERED TEST BASIS

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.3.15.1	---------- **- NOTE -** Only required to be met when all four cold leg temperatures are > 275°F. ---------- Verify pressurizer heater circuit breakers trip open on an actual or simulated actuation signal.	24 months
SR 3.3.15.2	Verify reactor coolant pump breakers trip open on an actual or simulated actuation signal.	24 months
SR 3.3.15.3	Verify main feedwater and startup feedwater pump breakers trip open on an actual or simulated actuation signal.	24 months
SR 3.3.15.4	---------- **- NOTE -** Only required to be met in MODES 1 and 2. ---------- Verify auxiliary spray and purification line isolation valves actuate to the isolation position on an actual or simulated actuation signal.	24 months

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition A not met during movement of irradiated fuel assemblies. OR One or more Functions within two or more required divisions inoperable during movement of irradiated fuel assemblies.	D.1 Suspend movement of irradiated fuel assemblies.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.3.16.1	---------- **- NOTE -** Only required to be met in MODE 5. ---------- Verify reactor coolant pump breakers trip open on an actual or simulated actuation signal.	24 months

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.3.16.2	---------- **- NOTES -** 1. Not required to be met in MODE 5 above the P-12 (Pressurizer Level) interlock. 2. Not required to be met in MODE 6 with water level $\geq$ 23 feet above the top of the reactor vessel flange. ---------- Verify CVS letdown isolation valves actuate to the isolation position on an actual or simulated actuation signal.	24 months

SURVEILLANCE REQUIREMENTS

- NOTE -
Refer to Table 3.3.17-1 to determine which SRs apply for each PAM Function.

	SURVEILLANCE	FREQUENCY
SR 3.3.17.1	**- NOTE -** Not required to be met for Neutron Flux (Intermdiate Range) in MODE 1. Perform CHANNEL CHECK for each required instrumentation channel that is normally energized.	31 days
SR 3.3.17.2	**- NOTE -** Neutron detectors are excluded from CHANNEL CALIBRATION. Perform CHANNEL CALIBRATION.	24 months

Table 3.3.17-1 (page 1 of 1)
Post-Accident Monitoring Instrumentation

	FUNCTION	REQUIRED CHANNELS	CONDITION REFERENCED FROM REQUIRED ACTION D.1	SURVEILLANCE REQUIREMENTS
1.	Neutron Flux (Intermediate Range)	2	E	SR 3.3.17.1 SR 3.3.17.2
2.	Reactor Coolant System (RCS) Hot Leg Temperature (Wide Range)	2	E	SR 3.3.17.2
3.	RCS Cold Leg Temperature (Wide Range)	2	E	SR 3.3.17.2
4.	RCS Pressure (Wide Range)	2	E	SR 3.3.17.2
5.	RCS Subcooling	2	E	SR 3.3.17.2
6.	Containment Water Level	2	E	SR 3.3.17.2
7.	Containment Pressure	2	E	SR 3.3.17.2
8.	Containment Pressure (Extended Range)	2	E	SR 3.3.17.2
9.	Containment Area Radiation (High Range)	2	E	SR 3.3.17.2
10.	Pressurizer Level and Associated Reference Leg Temperature	2	E	SR 3.3.17.2
11.	In-Containment Refueling Water Storage Tank (IRWST) Wide Range Water Level	2	E	SR 3.3.17.2
12.	Passive Residual Heat Removal (PRHR) Heat Removal	2	E	SR 3.3.17.1 SR 3.3.17.2
13.	Core Exit Temperature -- Quadrant 1	2[(a)]	E	SR 3.3.17.2
14.	Core Exit Temperature -- Quadrant 2	2[(a)]	E	SR 3.3.17.2
15.	Core Exit Temperature -- Quadrant 3	2[(a)]	E	SR 3.3.17.2
16.	Core Exit Temperature -- Quadrant 4	2[(a)]	E	SR 3.3.17.2
17.	Passive Containment Cooling System (PCS) Heat Removal	2	E	SR 3.3.17.1 SR 3.3.17.2
18.	Penetration Flow Path Remotely Operated Containment Isolation Valve Position	2 per penetration flow path[(b)(c)(d)]	E	SR 3.3.17.1 SR 3.3.17.2
19.	IRWST to Normal Residual Heat Removal System (RNS) Suction Valve Status	2	E	SR 3.3.17.1 SR 3.3.17.2
20.	Pressurizer Pressure	2	E	SR 3.3.17.2

(a) A channel consists of two thermocouples within a single division. Each quadrant contains two divisions. The minimum requirement is two OPERABLE thermocouples in each of the two divisions.

(b) Not required for isolation valves whose associated penetration is isolated by at least one closed and deactivated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.

(c) Only one position indication channel is required for penetration flow paths with only one installed control room indication channel.

(d) Penetration Flow Path Remotely Operated Containment Isolation Valve Position applies to components that receive the ESF containment isolation signal (T signal).

3.3 INSTRUMENTATION

3.3.19 Diverse Actuation System (DAS) Manual Controls

LCO 3.3.19 The DAS manual controls for each function in Table 3.3.19-1 shall be OPERABLE.

APPLICABILITY: According to Table 3.3.19-1.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or more manual DAS controls inoperable.	A.1 Restore DAS manual controls to OPERABLE status.	30 days
B. Required Action and associated Completion Time of Condition A not met for inoperable DAS manual reactor trip control.	B.1 Perform SR 3.3.7.1. AND	Once per 31 days on a STAGGERED TEST BASIS
	B.2 Restore all controls to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry
C. Required Action and associated Completion Time of Condition A not met for inoperable DAS manual actuation control other than reactor trip.	C.1 Restore all controls to OPERABLE status.	Prior to entering MODE 2 following next MODE 5 entry

SURVEILLANCE REQUIREMENTS

- NOTE -

Refer to Table 3.3.20-1 to determine which SRs apply for each ADS and IRWST Injection Blocking Device Function.

SURVEILLANCE		FREQUENCY
SR 3.3.20.1	Verify each ADS and IRWST Injection Block switch is in the “unblock” position.	7 days
SR 3.3.20.2	Perform CHANNEL CALIBRATION in accordance with Setpoint Program.	24 months
SR 3.3.20.3	Perform ACTUATION LOGIC TEST of ADS and IRWST Injection Blocking Devices.	24 months
SR 3.3.20.4	--- **- NOTE -** Verification of setpoint not required. --- Perform TRIP ACTUATING DEVICE OPERATIONAL TEST (TADOT) of ADS and IRWST Injection Block manual switches.	 24 months
SR 3.3.20.5	The following SRs of Specification 3.5.2, “Core Makeup Tanks (CMTs) – Operating” are applicable for each CMT: SR 3.5.2.3 SR 3.5.2.6 SR 3.5.2.7	In accordance with applicable SRs

Table 3.3.20-1 (page 1 of 1)
ADS and IRWST Injection Blocking Device

	FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS PER DIVISION	SURVEILLANCE REQUIREMENTS
1.	Core Makeup Tank Level for Automatic Unblocking[a]	1,2,3,4[b]	2	SR 3.3.20.2 SR 3.3.20.3 SR 3.3.20.5
2.	ADS and IRWST Injection Block Switches for Manual Unblocking	1,2,3,4[b]	1	SR 3.3.20.3 SR 3.3.20.4
		4[c],5,6	1	SR 3.3.20.1 SR 3.3.20.3 SR 3.3.20.4

(a) Not required to be OPERABLE with associated divisional ADS and IRWST Injection Block switch in the “unblock” position.

(b) With the Reactor Coolant System (RCS) not being cooled by the Normal Residual Heat Removal System (RNS).

(c) With the RCS being cooled by the RNS.

5.5 Programs and Manuals

5.5.14 Setpoint Program (SP)

a. The Setpoint Program (SP) implements the regulatory requirement of 10 CFR 50.36(c)(1)(ii)(A) that technical specifications will include items in the category of limiting safety system settings (LSSS), which are settings for automatic protective devices related to those variables having significant safety functions.

b. The Nominal Trip Setpoint (NTS), As-Found Tolerance (AFT), and As-Left Tolerance (ALT) for each Technical Specification required automatic protection instrumentation function shall be calculated in conformance with WCAP-16361-P, “Westinghouse Setpoint Methodology for Protection Systems – AP1000,” February 2011.

c. For each Technical Specification required automatic protection instrumentation function, performance of a CHANNEL CALIBRATION surveillance “in accordance with the Setpoint Program” shall include the following:

1. The as-found value of the instrument channel trip setting shall be compared with the previously recorded as-left value.

i. If the as-found value of the instrument channel trip setting differs from the previously recorded as-left value by more than the pre-defined test acceptance criteria band (i.e., the specified AFT), then the instrument channel shall be evaluated to verify that it is functioning in accordance with its design basis before declaring the surveillance requirement met and returning the instrument channel to service. An Instrument Channel is determined to be functioning in accordance with its design basis if it can be set to within the ALT. This as-found condition shall be entered into the plant’s corrective action program.

ii. If the as-found value of the instrument channel trip setting is less conservative than the specified AFT, the surveillance requirement is not met and the instrument channel shall be immediately declared inoperable.