

UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

THE ATOMIC SAFETY AND LICENSING BOARD

Robert M. Lazo, Esquire, Chairman  
Dr. Hugh C. Paxton, Member  
Dr. Paul W. Purdom, Member



In the Matter of  
JERSEY CENTRAL POWER & LIGHT COMPANY  
(Oyster Creek Nuclear Generating  
Station)

Docket No. 50-219  
(Conversion to Full-Term  
Operating License Proceedings)  
February 22, 1980

ORDER DISMISSING PROCEEDING

On May 31, 1979, the Atomic Safety and Licensing Board issued a Memorandum and Order which approved the environmental technical specifications previously submitted for the Board's review and which indicated that the pending Motion to Terminate Hearing could now be considered except for one concern.

The Board asked that the NRC Staff advise it of the significance of the recent (May 2, 1979) loss of coolant inventory transient and consequent plant shutdown at Oyster Creek Nuclear Generating Station.

Thereafter, on June 8, 1979, the Staff provided the Board with a copy of the Restart Safety Evaluation Report (Restart SER) concerning the event which the Staff had issued on May 30, 1979. As indicated by the Restart SER, the Staff concluded the Oyster Creek core was undamaged and that with additional technical specifications imposed there was reasonable

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assurance plant operation could be resumed without undue risk to the health and safety of the public. The Licensee was authorized to resume operation on May 30, 1979. On September 25, 1979, the Staff again moved to terminate the proceeding. Having reviewed the information provided by the Staff, the Board has concluded it has no further questions to be addressed in this matter and the Staff requests that the proceeding be terminated should be granted.

The description of the abnormal occurrence event and the remedial actions taken by the Licensee and the Staff are summarized in the report published by the Commission in the Federal Register on August 30, 1979 (44 Fed. Reg. 50925-28).

The loss of feedwater transient at the Oyster Creek facility on May 2, 1979, resulted in a significant reduction in water inventory above the reactor core area as measured by one set of water level instruments (triple-low level), while the remaining two sets of level instrumentation in the reactor annulus indicated water levels above any protective feature setpoint. The water level measured within the core shroud area fell below the triple-low level setpoint, a safety limit, of five feet six inches above the top of the fuel. Subsequent analyses by the licensee have conservatively determined that the minimum water level over the top of the fuel was one and one-half feet. Coolant sample analyses and offgas release rates support the conclusion that no fuel damage occurred.

After performing a thorough evaluation of the event to determine whether any fuel damage had occurred and discussing the evaluation with the Staff, the Licensee took the following actions:

1. The triple-low level was established as a Safety Limit for all modes of reactor operation;
2. A requirement was added to the Technical Specifications that the suction and discharge valves in at least two recirculation loops be open at all times. The procedures were changed to implement this requirement;
3. Operator training sessions were held, the event was thoroughly discussed and the revised procedures reviewed.

Following Staff review and preparation of the Safety Evaluation Report, the following three requirements were added to the Technical Specifications:

1. The triple-low level was made a Safety Limit for all mode-switch positions;
2. At least two recirculation loop discharge and suction valves must remain in the full open position;
3. The time duration of the low-low level signal was required to be not greater than that used in the safety analysis for the limiting loss-of-inventory transient.

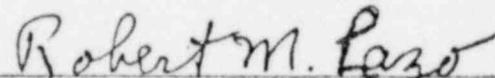
It was concluded that no evidence of fuel damage was apparent and that the facility could be safely returned to operation.

The Joint Motion to terminate the proceeding is granted. All proceedings involving the application to convert the provisional operating license for this facility to a full-term operating license are terminated and the case is dismissed.

Dr. Hugh C. Paxton and Dr. Paul W. Purdom, Board Members, join in and concur with this Order.

It is so ORDERED.

FOR THE ATOMIC SAFETY AND  
LICENSING BOARD

  
Robert M. Lazo, Chairman

Dated at Bethesda, Maryland,  
this 22nd day of February, 1980.