

CONTROL BLOCK:

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 (1)

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60

0 1 N C B E P 2 2 0 0 - 0 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

REPORT SOURCE: 01 L 6 0 5 0 - 0 3 2 4 7 0 1 1 4 8 0 8 0 2 1 1 0 0 9

POCKET NUMBER: 80 61 58 59 74 75 80

EVENT DATE: 01 11 48 08

REPORT DATE: 02 11 00 09

0 2 During normal plant operation, the nuclear engineer performed an OD-1 which determined
0 3 that MCPR was 1.281 for bundle 43-28, an 8x8R bundle. The reportable limit for this
0 4 parameter is < 1.29. Power was being slowly increased following a rod shuffle the
0 5 night before.

7 8 9 30

SYSTEM CODE R B 11		CAUSE CODE E 12		CAUSE SUBCODE E 13		COMPONENT CODE I N S T R U 14		COMP. SUBCODE E 15		VALVE SUBCODE Z 16	
LER NO REPORT NUMBER 17		EVENT YEAR 8 0 21 22		SEQUENTIAL REPORT NO. 0 1 0 24 25 26		OCCURRENCE CODE 0 3 28 29		REPORT TYPE L 30		REVISION NO. 0 32	
ACTION TAKEN X 18		FUTURE ACTION X 19		EFFECT ON PLANT B 20		SHUTDOWN METHOD Z 21		HOURS 0 0 0 4 22 37 40		ATTACHMENT SUBMITTED Y 23	
NPRD-4 FORM SUB. Y 24		PRIME COMP. SUPPLIER N 25		COMPONENT MANUFACTURER 6 0 8 0 26							

1 0 The CPR problem was due to: a) the controlling LPRM had two levels bypassed;

1 1 b) problems with the core flow input. Power was immediately reduced to bring MCPR in

1 2 specifications. The LPRM will be replaced and the inoperable jet pump input will be

1 3 corrected during the next refueling outage. A "direct flow" vs. "core flow" corre-

1 4 lation has been inserted into the computer to provide more representative indication

8 9 FACILITY STATUS (28) % POWER (29) OTHER STATUS (30) METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)

1 5 E 0 9 5 NA B Routine OD-1

7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

ACTIVITY CONTENT
RELEASED OF RELEASE

1 6 2 33 34

AMOUNT OF ACTIVITY (35)

NA

LOCATION OF RELEASE (36)

NA

PERSONNEL EXPOSURES			
NUMBER		TYPE	DESCRIPTION
1	7	0 0 0	(37) Z (38) NA

PERSONNEL INJURIES		DESCRIPTION	
NUMBER			
1	R	000	40 NA

7	8	9	11	12			8002200	685	80
		LOSS OF OR DAMAGE TO FACILITY		(43)					
		TYPE DESCRIPTION							
1	9	2	(42)		NA				

PUBLICITY DESCRIPTION (45) NA

NRC USE ONLY

PHONE: 919-457-9521

LER CONTINUATION -- RO# 2-80-10

Facility: BSEP Unit # 2

Event Date 1-14-80

of core flow. Had the proper core flow been seen by the computer, it is believed that this event would not have exceeded any limits.