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EXEMPT _____

DW. 50-244

MEMORANDUM FOR: E. L. Jordan, Director, Division of Engineering and Quality Assurance, IE

FROM: W. R. Mills, Acting Chief, Events Analysis Branch, Division of Engineering and Quality Assurance, IE

SUBJECT: STEAM GENERATOR TUBE RUPTURE AT GINNA ON JANUARY 25, 1982

This memorandum provides a compilation of preliminary information concerning the steam generator tube rupture event at Ginna on January 25, 1982.

1. Chronology of Notification - The times and methods by which notifications were made are shown in Enclosure 1. The more salient features of the notifications show that: (1) the licensee called the NRC Operations Center via the direct telephone line within approximately 8 minutes of the first indications of steam generator tube failure and 5 minutes of the resulting reactor trip; (2) the direct telephone line from the site was continuously manned by a licensee employee or the NRC resident inspector until a controlled cooldown was assured; (3) the NRC Regional Office was notified immediately after the licensee called the NRC Operations Center; and (4) the Regional and Headquarters Offices manned their response centers in a timely manner after which the Region dispatched a team to the site.

2. Reactor Sequence of Events - Preliminary information on the equipment performance and plant staff response to the steam generator tube rupture is described in Enclosure 2.

3. Radiological Considerations - Preliminary information on the radiological chronology and discussions of radioactive releases are provided in Enclosure 3.

4. Other Documentation - Copies of the PN's are provided in Enclosure 4 for convenience.

The preliminary information presented in these enclosures was compiled during and shortly after the event. It has been informally distributed to various

CONTACT: I. Villalva, IE
49-24865

OFFICE							
SURNAME							
DATE							

NRC personnel. This information will be supplemented by the results of ongoing NRC evaluations.

W. R. Mills, Acting Chief
Events Analysis Branch
Division of Engineering and
Quality Assurance, IE

Enclosures: As stated

cc/w enclosures:
R. C. DeYoung, IE
J. H. Sniezek, IE
H. R. Denton, NRR
E. G. Case, NRR
D. G. Eisenhut, NRR
R. H. Vollmer, NRR
R. J. Mattson, NRR
J. J. Kramer, NRR
S. H. Hanauer, NRR
V. Stello, DEDROGR
C. Michelson, AEOD
J. Heltemes, AEOD
J. R. Shea, IP
J. D. LaFleur, IP
W. Kerr, SP
J. G. Davis, NMSS
R. A. Purple, NRR
T. M. Novak, NRR
G. Lainas, NRR
T. A. Ippolito, NRR
R. B. Minogue, RES
D. Ross, RES
R. C. Haynes, RI
J. P. O'Reilly, RII
J. G. Keppler, RIII
J. T. Collins, RIV
R. H. Engelken, RV
R. W. Starostecki, RI
T. T. Martin, RI
G. H. Smith, RI
R. C. Lewis, RII
J. Olshinski, RII
J. Stohr, RII
R. L. Spessard, RIII
C. E. Norelius, RIII
J. Hind, RIII
J. E. Gagliardo, RIV
J. Crews, RV
G. Spencer, RV

OFFICE	EAB:DE&QA:IE	CEAB:DE&QA:IE					
SURNAME	IVillalva	WRMills					
DATE	2/1/82:jr	2/ /82					

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY * USGPO: 1980—329-824

CRITICAL OPERATIONAL EVENTS AT NRC OC
GINNA EVENT
1/25-26/82

0933 to 0937 . Shift Supervisor at Ginna called NRC HQ with a Notification of Unusual Event. Ginna decided to keep line open.

0938 . HDO established telecon with Ginna and Region I, then notified IE and NRR personnel.

0949 . HDO notified FEMA.

0952 . Region I began to mobilize personnel at their IRC.

0953 . IE, Region I, and Ginna CR and TSC personnel join in telephone conference.

1030 . OC activiated after licensee declaration of Alert. Response personnel selected.

1035 . Chairman's Office notified.

1035 to 1055 . Selected response personnel called to report to OC. Key personnel notified in Public Affairs, State Programs, NMSS.

1040 . Executive Team and Operations Team personnel begin to arrive at OC.

1045 . Licensee declared Site Area Emergency.

1100 . Haynes in conference call with Executive Team.

1125 . Chairman arrived at OC, then fully manned.

1125 to 1150 . Other agencies notified: DOE, HHS, EPA, White House Situation Room.

1130 . NRC Region I and FEMA personnel leave for site.

1145 to 1205 . Additional NRC offices notified: Commissioners (through Chairman's office), Security, Congressional Affairs.

1155 and 1330 . Chairman briefed Governor Carey.

1520 . NRC released DOE personnel (RAT at Brookhaven and ARMS at Andrews AFB) being held in standby.

1525 . Chairman appointed EDO as Acting Director and left OC.

. Region I team, under Haynes, arrived at TSC.

1700 . After briefing with Acting Director, Haynes appointed Director of Site Operations with full authority; Headquarters team established to support DSO.

1710 to 1740 . Governor Carey, Chairman and other Commissioners, and other agencies notified of transfer of authority to DSO.

2015 . Licensee downgraded status to Alert; Headquarters support team on midnight shift reduced.

January 26

1115 . Licensee ended emergency status and entered recovery mode.

1500 . EOF deactivated.

1600 . NRC response retured to Normal mode but personnel remained at Ginna. Headquarters confirmed status with personnel in TSC.

1625 to 1700 . Final status calls made to Chairman, other Commissioners, and other Federal agencies contacted by Headquarters during the response.

GINNA JANUARY 25, 1982 STEAM GENERATOR TUBE RUPTURE EVENT

SEQUENCE OF EVENTS
(TIMES E.S.T.)

JANUARY 25:

0925	Charging Pump Speed Alarm; "B" Steam Generator (SG) level alarm; Steam flow - feedwater flow to SG-B mismatch; Steam jet air ejector (SJAE) radiation monitor alarm; low pressurizer pressure (2170 psig).
0928	Reactor trip on low pressure; Automatic safety injection (SI) actuated; Containment isolated.
0929	Pressurizer level offscale low; Reactor Coolant System (RCS) pressure approximately 1200 psig.
0933	Both reactor coolant pumps (RCPs) manually tripped.
0940	Main steam isolation valve for SG-B manually closed following indication of RCS leakage into SG-B (increasing level and pressure in SG-B).
0953	SG-A pressure 540 psig and level 76%; SG-B pressure 825 psig and level 89%. Plant being cooled by dumping steam from SG-A to main condensor while relying on natural circulation in loop "A". Subcooling being maintained at about 70°F; SI pumps on.
0957	SI circuitry reset; Instrument air for controlling containment isolation valves restored.
1000	Radiation level at SJAE reading 15000 counts and trending down. Was full scale 5 to 10 minutes previously.

JANUARY 25 (Continued)

1003 SG-A level reading 100% on narrow range and 400" on wide range meters.

1004 Charging pumps restarted; RCS pressure 1300 psig; Pressurizer level 18%.

1007 Pressurizer power operated relief valve (PORV) manually cycled to reduce RCS pressure per Station Procedures to reduce leak rate; Pressurizer level 10%.

1008 Pressurizer PORV manually cycled again.

1009 Pressurizer PORV manually opened, unable to shut PORV; Pressurizer pressure dropped from 1300 to 800 psig; Pressurizer level increasing; Pressurizer Relief Tank (PRT) high temperature alarm; Pressurizer PORV Block valve shut. Pressurizer level offscale high. First indication of a steam bubble in the Reactor Vessel Head.

1010 SI pumps increase RCS pressure to 1300 psig.

1018 Incore thermocouples indicate 458 degrees.

1025 SG-B atmospheric relief (PORV) manually isolated as a precaution.

1031 Reactor Vessel Head temperature 525° (by thermocouple).

1040 SG-B code safeties lifting (setpoint 1085 psig); Safety Injection pumps secured to reseat safeties; all charging pumps operating.

1050 Reactor Coolant Pump seal return relief lifted as a result of earlier containment isolation; Seal return relief discharged to the PRT.

JANUARY 25 (Continued)

1057 PRT rupture discs ruptured releasing RCS water to the "A" containment Sump.

1115 One Safety Injection Pump started; SG-B safeties lifted; RCS pressure at 1035 psig.

1129 "A" Reactor Coolant Pump restarted.

1200 Bubble drawn in Pressurizer; Pressurizer level at 80%.

1205 Established normal letdown.

1230 Cooling down at 2 degrees per hour by dumping steam from SG-A through the atmospheric PORV (condenser lost - SJAE not functional); RCS Press=923 psig.

1400 Containment Sump "A" at 9.3 feet (approximately 8000 gal); PRT at 92%.

1700 NRC Region I Incident Response Team onsite.

1840 Reestablished level in SG-B. Plant cooling down via single loop circulation dumping steam from SG-A to atmosphere. SG-B being cooled by feeding AFW and bleeding via the ruptured tube(s) to the RCS.

JANUARY 26

0705 RHR initiated with "A" Reactor Coolant Pump running to promote backflow circulation through the "B" loop; RCS pressure 280 psig. RCS temperature 330° (TC and core thermocouples agree). Maintaining temperature and pressure from about 0705 to 1200 (no net cooling) to degass RCS.

JANUARY 26 (Continued)

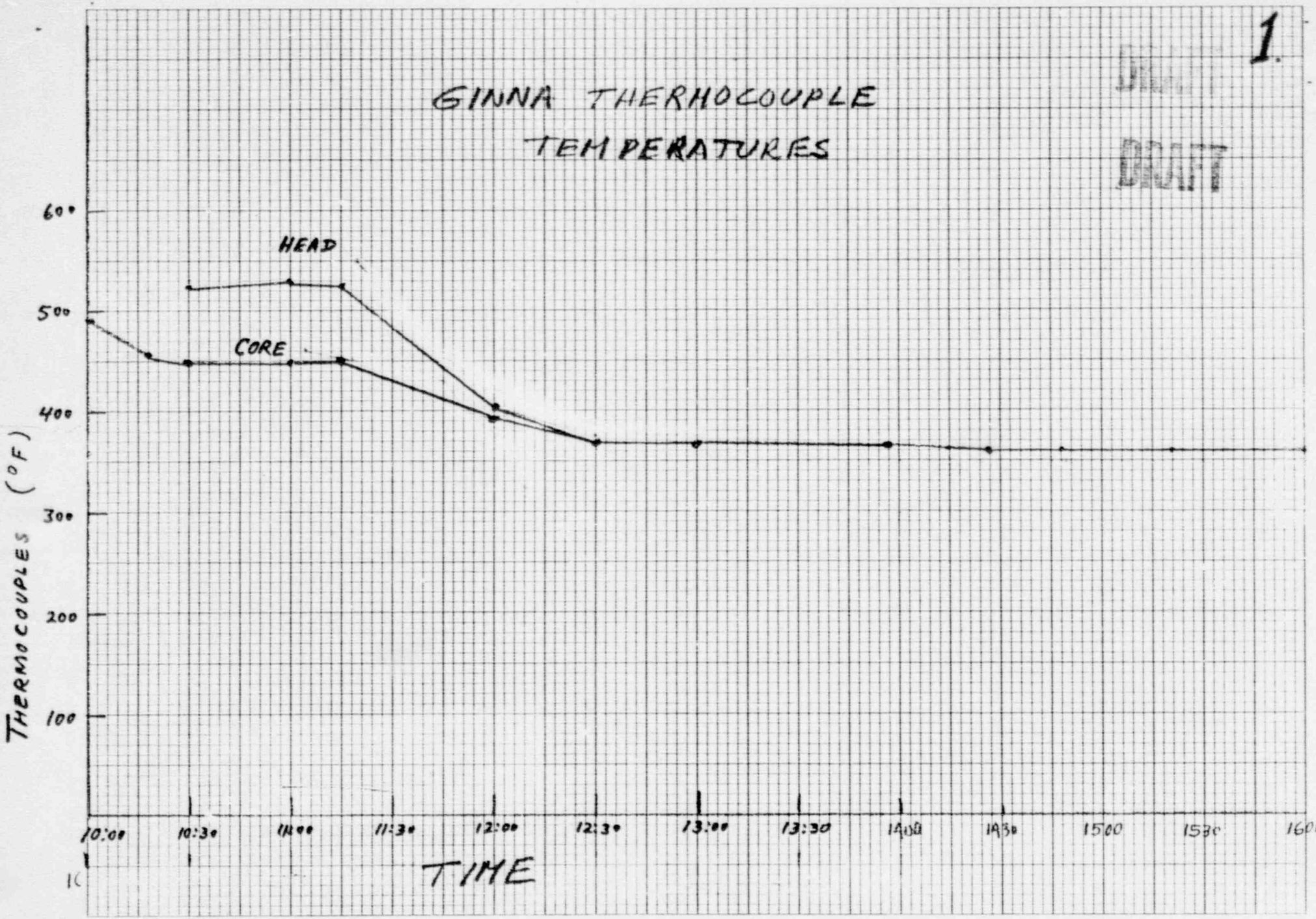
1300 Sometime between 1200 and 1300 RCS cooldown started at a rate of approximately 50°F per hour, after which cooldown rate was lowered to about 30°F per hour.

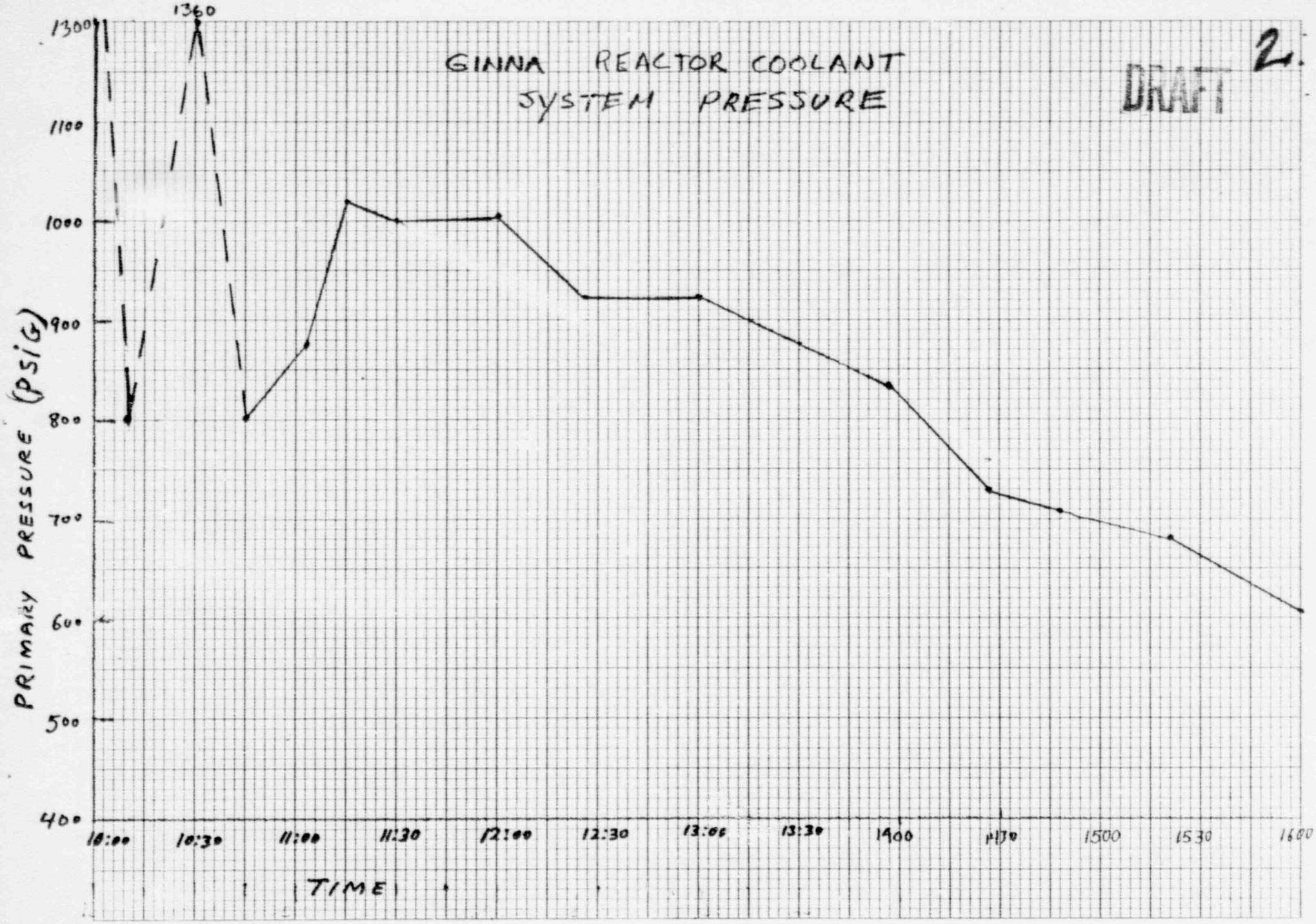
Graphs plotting the preliminary data received at the NRC response center are appended hereto.

Time	Incore Thermo Couple °F	Head Thermo Couple °F	A Loop Cold Leg °F	B Loop Cold Leg °F	RCS Press. psig	Pressurizer Level %	A S/G Press. psig	A S/G Level %	B S/G Press.
0626	341		344		297	50	85	48	321
0650	338		342		285	51	82	46	310
0726	327		331		278	37	70	51	307
0805	330		326		275	50	69	54	302
0833	326		330		276	35	69	53	301
0856	326		329		262	36	69	53	294
0924	326		330		257	36	69	52	289
0953	326		330		259	30	69	51	285
1028	326		331		254	30	70	50	282
1058	325		329		248	32	68	50	277
1307	297		299		225	37	58	46	254
1357	267		270		234	41	46	44	252
1459	240		243		201	38	35	43	240

Time	Incore Thermo Couple °F	Head Thermo Couple °F	A Loop Cold Leg °F	B Loop Cold Leg °F	RCS Press. psig	Pressurizer Level %	A S/G Press. psig	A S/G Level %	B S/G Press.
0933			540		1300				
1005	492		476	345	1300	18		100	
1009			480	350	1050				
1018	458		460	360					
1023						100 +			
1032	450	525			1360				
1043	450				790				
1048		525			800				
1054	450	529			936				
1105		525			1035		310		
1115	450				1020				
1150	408	416	405		984	100 +	207		872
1159			400		1006		195		
1200	395	400	380		1006	100 +	195		870
1203						98			
1225					974	60	184		
1227		370	375		923	62	131		
1231	374		374		923	67	110		854
1255	371	369	374		944	67	131		
1307	368		373		922	67	130		
1332	368	368	373		877	56	130		
1355	367		371		835	47	130		860
1425	361		361		729	52	120		751
1448	361	362	364		709	51			

Time	Incore Thermo Couple °F	Head Thermo Couple °F	A Loop Cold Leg °F	B Loop Cold Leg °F	RCS Press. psig	Pressurizer Level %	A S/G Press. psig	A S/G Level %	B S/G Press.
1520	360		364		679	50	117		700
1539	358				629	46	115		650
1600	358	357	360		609	51	112		633
1857	355		357		486	54	105		521
1928	353		356		476	47	103	49	505
1958	354		356		449	57	108	35	493
2026	352		356		459	53	103	39	491
2056	351		353		457	45	99	43	479
2220	345		349		438	55	93	23	472
2249	343		346		434	56	89	31	476
2322	339		342		413	56	81	32	453
2351	339		342		393	53	84	42	426
0021	339		345		362	60	84	41	411
0049	339		345		362	51	86	47	398
0116	339		346		370	44	89	43	383
0149	344		347		350	56	90	35	371
0219	344		347		346	66	91	32	364
0253	342		346		334	57	87	37	357
0328	341		345		326	56	85	43	350
0352	340		344		319	58	83	47	343
0421	340		345		318	54	86	44	342
0458	341		344		309	51	85	46	334
0526	341		344		303	50	85	47	330
0554	340		343		301	51	84	49	324





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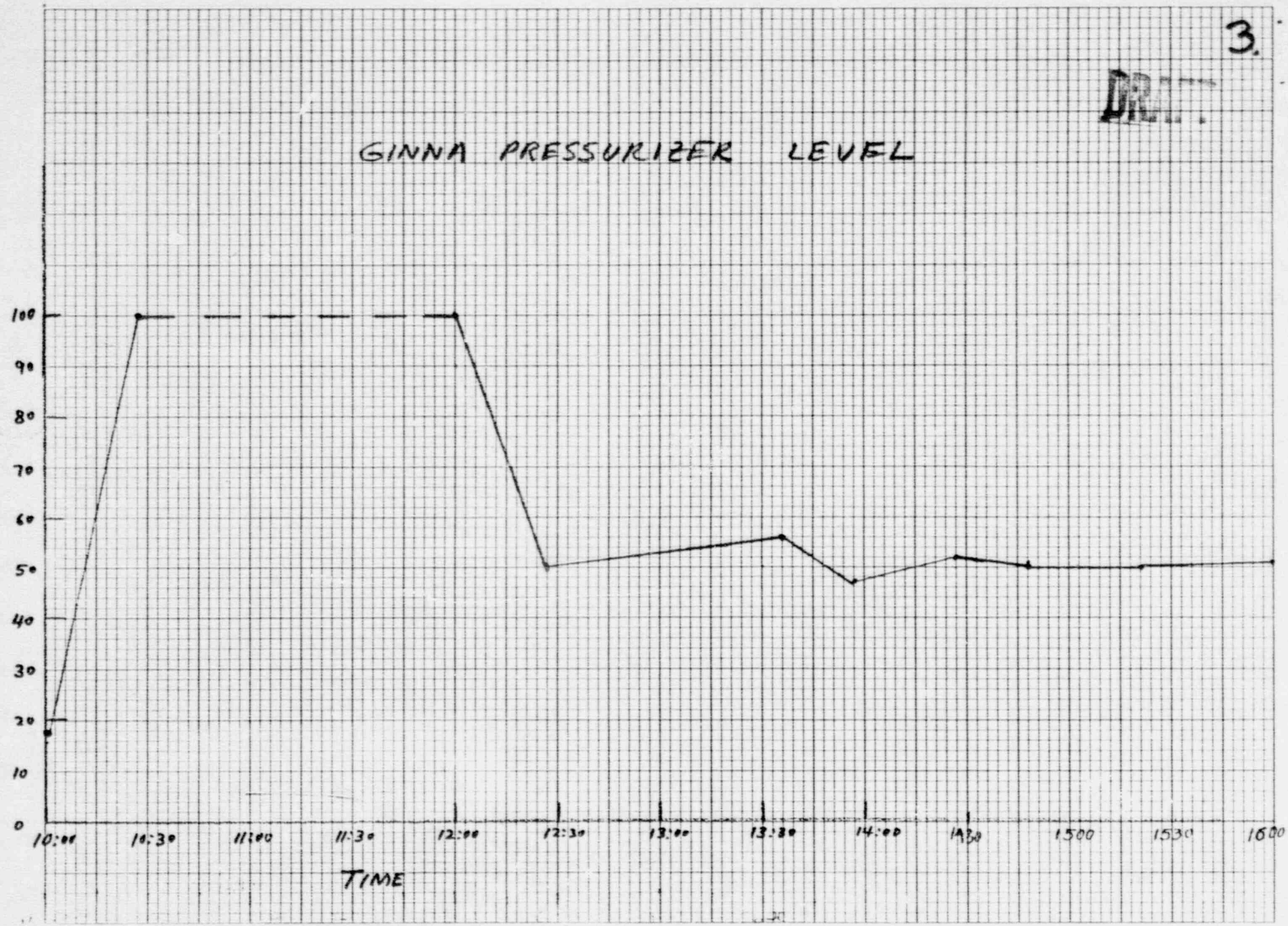
GINNA PRESSURIZER LEVEL



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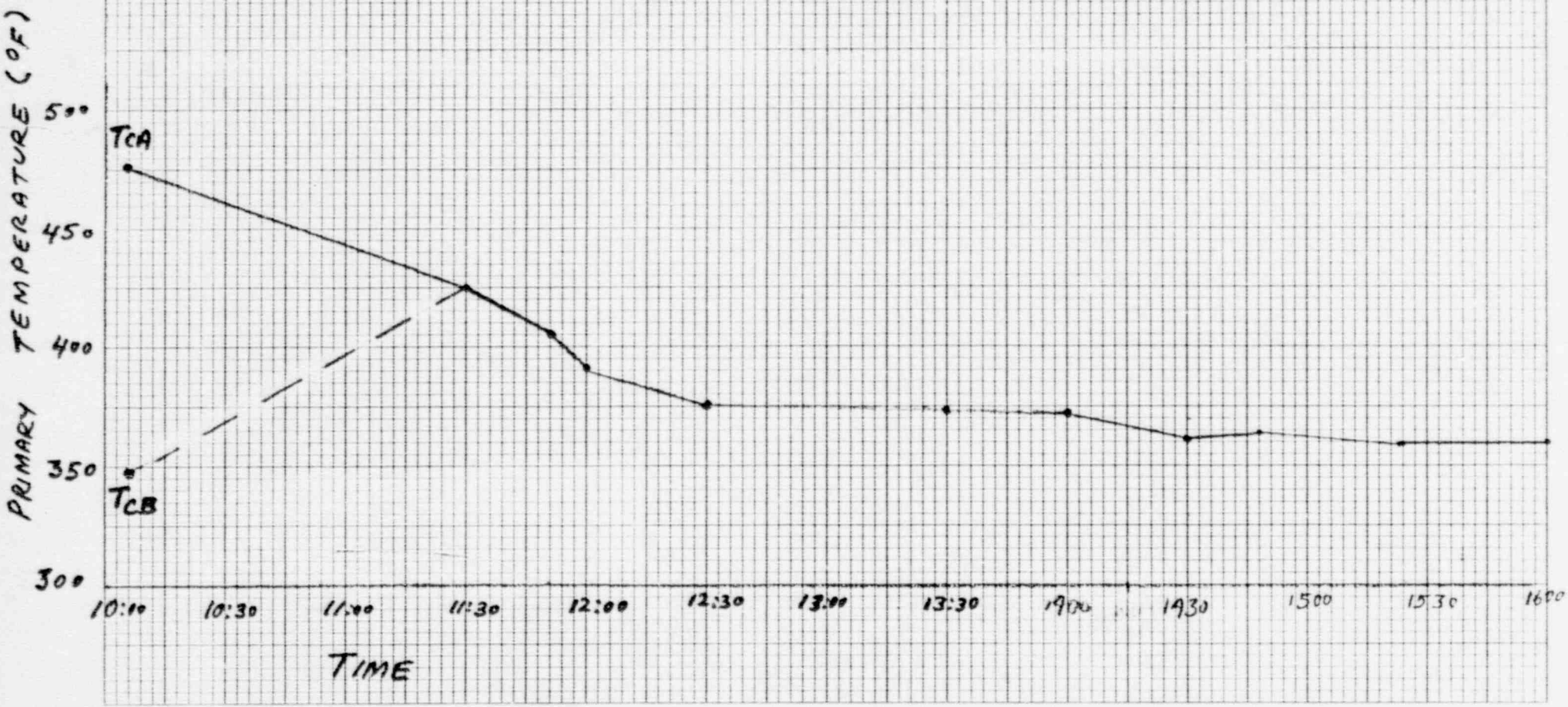
GINNA RCS COLD LEG TEMPEATURE



Enclosure 3

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RAD CHRONOLOGY

January 25

1015 Environmental measurements team sent out by licensee.

1044 Environmental team reports 10 - 12 mR/hr at Training Center - believed to be peak dose rate.

1045 Site emergency declared.

1058 Met conditions - wind from 330° at 20 mph (gusting winds).

1106 Environmental team reports 1.2 mR/r at 1 mile SSE of plant.

1115 (approx) Containment radiation level _ 6.7 mR/hr (normal level 2.5 mR/hr)

1115 Environmental team reports 0.8 mR/hr at 1 mile SE of plant (believed to be same location as 1106 reading).

1130 (approx) Plant began steam dump to atmosphere from steam generator A.

1200 Environ. team reports 0.2 mR/hr at Training Center.

1200 (approx) Environ. team reports 3 mR/hr near plant to the South (under or in visible steam cloud).

1200 Met conditions - wind from 317° at 11 mph.

1250 Environ. team reports background dose rates at Training Center.

1250 (approx)

- o Containment radiation level - 7.8 mR/hr.
- o Area monitors in auxiliary bldg. reading normal (10 mR/hr).
- o Containment air monitor reads 13,000 cpm (5000 cpm - normal).
- o Environ. team reports all dose rates levels at background out to 1 mile.
- o Degassed primary coolant sample analyzed -
  - 2250 ppm - boron
  - 0.32 uCi/gm - gross degassed activity
  - I-131 Dose Equivalent 2.5(-2)
  - I-131 - 7.6 (-3) uCi/gm
  - I-132 - 1.1 (-1) uCi/gm
  - I-133 - 8.8 (-2) uCi/gm
  - I-134 - 4.1 (-2) uCi/gm
  - I-135 - 3.3 (-2) uCi/gm
- o Secondary water sample ("B") steam generator)
  - I-131 - 2.0 (-3)
  - I-132 - 3.9 (-2)
  - I-133 - 2.3 (-2)
  - I-134 - 2.2 (-2)

I-135 - 2.5 (-2)

- o Detectable levels of radioiodine in snow at 500 - 600 ft. from plant.

1310 Results of snow sample taken near Training Center. (Sample taken at 1210; analyzed at 1310 hrs.). This believed to be the highest reading

NOT CORRECTED FOR DECAY

1345
- o Environ. team reports no airborne activity offsite.
- o Main steam line reading 30 mR/hr.
- o Turbine building reading 3-4 mR/hr.
- o Highest offsite deposition on snow (beyond location of earlier 12 mR/hr) reported about 10,000 cpm.
- o Met Conditions: wind from 250°, 10 mph, stable conditions.

SNOW SAMPLE ANALYSIS

ISOTOPE	uCi/LITER*
I-131	0.04
I-132	0.24
I-133	0.35
I-134	0.06
I-135	0.47
Mo-99	0.006(??)

*uCi per liter of melted liquid

1539 Reading taken onsite east of Training Center parking lot shows 0.7 mr/hr at waist level and 5000 cpm at 1 inch above ground due to deposition.
Direct reading at junction of Lake Road and road to plant shows 0.1 mR/hr.

1606 Met conditions are as follows:
wind from 265° at 4.5 mph at the 33 ft. level neutral conditions, light snow, temp. 13.5°F.

1630 Rad monitoring Van from Region I departs for Site. ETA about 8 hrs.

1705 All offsite dose rates reading background levels.

Rad levels in plant are normal. "A" Steam Gen. reading 1×10^{-6} uCi/ml iodine activity.

Slight contamination reported in water puddles in Turbine Bldg.

2004 Met data from tower

LEVEL	TEMP.	WIND SPEED	WIND DIREC.
33 ft. level	14°F	14.5 mph	261°
150 ft. level	13.2°F	18.2 mph	248°
250 ft. level	13.2°F	0(?)	245°

2015 Status changed from Site Emergency to Alert.

2310 No significant offsite readings.

Ginna estimates that for a maximum of 14 minutes, they exceeded their instantaneous release rate by a factor of 9.6 times. This estimate is incomplete, because it does not include steam lost subsequently from the "B" loop.

January 26 0105 Met data from tower: Neutral Stability

LEVEL	TEMP.	WIND SPEED	WIND DIREC.
33 ft	14.7°F	8.5 mph	402 (??)
150 ft	14.0°F	25.5 mph	319°
250 ft	13.7°F	28.0 mph	308°

0405 HP Status Update
(no releases offsite. Normal readings in-plant).

0705 HP Status Update
(no releases offsite, normal readings in-plant).

- o Largest activity from a smear (wipe) sample on an automobile in on-site parking lot - 17,000 DPM/100 cm^2. (A typical surface contamination allowable level is about 1000 DPM/100 cm^2 in the plant).

- o Licensee is performing calculations to estimate total release and arrive at estimate of integrated dose offsite.

- o NRC RAD specialists performing independent calculations for offsite dose assessment.

5

GINNA STEAM GENERATOR PRESSURES

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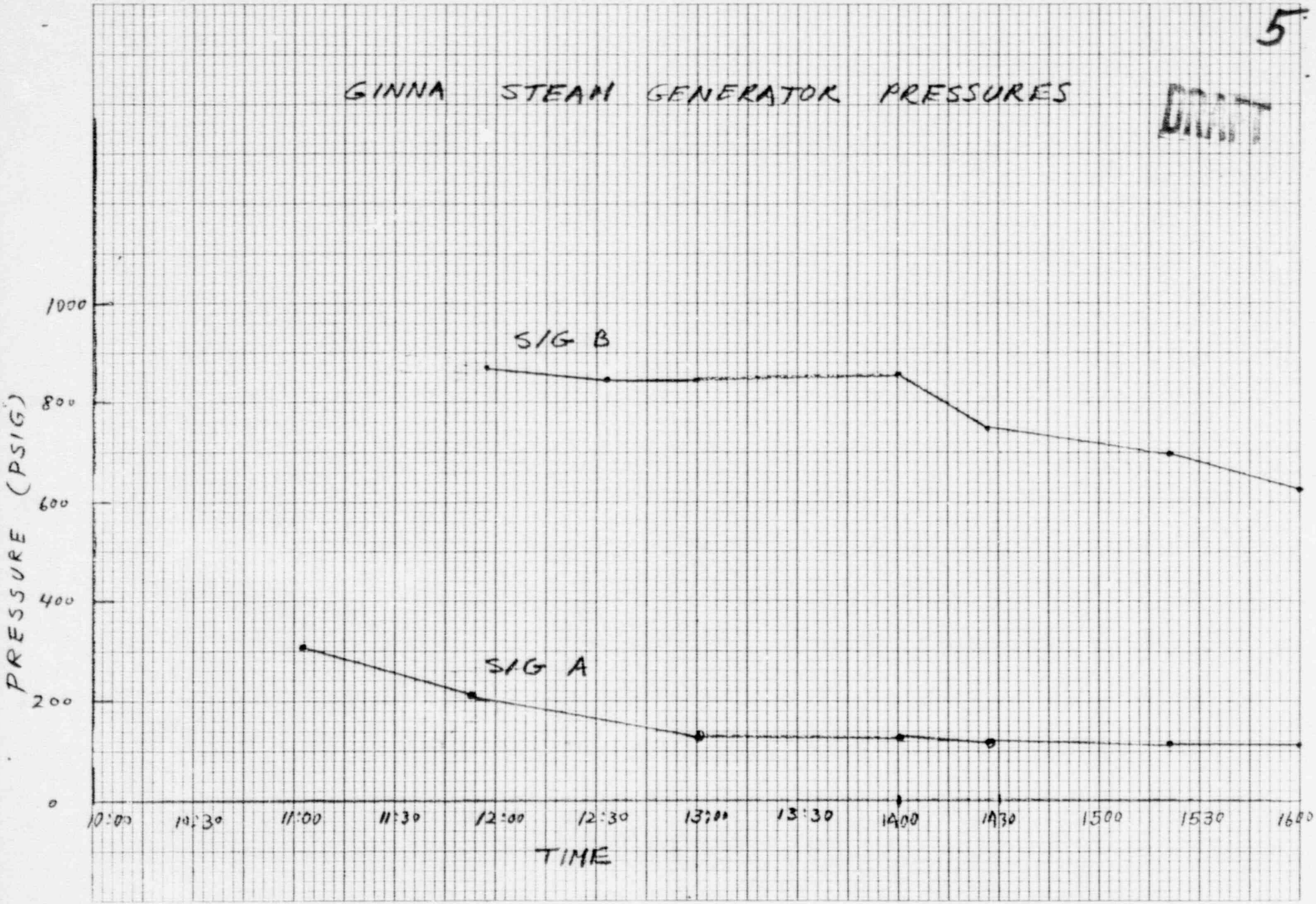

Date: January 26, 1982

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE--PNO-I-82-09B

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by IE staff on this date.

Facility: R.E. Ginna Nuclear Power Plant
DN 50-244
Ontario, New York

Licensee Emergency Classification:
_____ Notification of Unusual Event
__X__ Alert
_____ Site Area Emergency
_____ General Emergency
_____ Not Applicable

Subject: STEAM GENERATOR TUBE RUPTURE; SITE EMERGENCY

The Site Emergency was downgraded to an Alert at 7:15 p.m., January 25, 1982.

Cooldown of the Residual Heat Removal (RHR) system began about 7:00 a.m., January 26, 1982. At 8:00 a.m., the primary system was at 275 psig, 330 degrees. One charging pump is operating. There is normal letdown flow.

Reactor coolant samples indicate no fuel failures. Surveys indicate no radiation levels above background. The NRC Radiological Measurements Van should arrive about 9:00 a.m., January 26, 1982, to independently check licensee samples.

A backup response team from Region I has reinforced the onsite NRC contingent.

CONTACT: KISTER 488-1233 ALLAN 488-1340

DISTRIBUTION:

H. St._____	MNBB_____	Phillips_____	E/W_____	Willste_____	Landow (Faxed by HQ)
Chairman Palladino	EDO	NRR	IE	NMSS	OIA
Comm. Gilinsky	AEOD			RES	
Comm. Bradford	PA	Air Rights_____			MAIL:
Comm. Ahearne	MPA	SP		INPO_____	IE:TAS
Comm. Roberts	ELD			NSAC_____	ADM:Doc. Mgt. Br.
ACRS					
SECY	Regional Offices _____			TMI Resident Section _____	
CA				RI Resident Office _____	

Region I Form 83
(Rev. January 5, 1982)
Destroyed previous edition

Dcts
8202020375

DCS No: 50244-820125
Date: January 27, 1982

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE--PNO-I-82-09C

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by IE staff on this date.

Facility: R. E. Ginna Nuclear Power Plant
DN 50-244
Ontario, New York

Licensee Emergency Classification:
_____ Notification of Unusual Event
_____ Alert
__X__ Site Area Emergency
_____ General Emergency
_____ Not Applicable

Subject: STEAM GENERATOR TUBE RUPTURE; SITE EMERGENCY

At 7:05 a.m. on January 26, 1982, the Residual Heat Removal System was placed in service. The licensee downgraded the "Alert" to the "Recovery Mode" at 10:45 a.m. in accordance with the provisions of the emergency plan. The plant was placed in cold shutdown mode at 6:53 p.m.

The containment sump level previously reported at 9.3 feet (8000 gal) has been determined to be in error. Actual level is 5.3 feet (about 1,900 gal). The initial level reported was based on the higher of the two level indication channels. The lower reading channel was determined to be correct.

The NRC Region 1 evaluation is continuing.

CONTACT: Kister 488-1233 Allan 488-1340

DISTRIBUTION:

H. St._____	MNBB_____	Phillips_____	E/W_____	Willste_____	Landow (Faxed by HQ)
Chairman Palladino	EDO	NRR	IE	NMSS	OIA
Comm. Gilinsky	AEOD			RES	
Comm. Bradford	PA	Air Rights_____			MAIL:
Comm. Ahearne	MPA	SP		INPO_____	IE:TAS
Comm. Roberts	ELD			NSAC_____	ADM:Doc. Mgt. Br.
ACRS					
SECY	Regional Offices _____			TMI Resident Section _____	
CA				RI Resident Office _____	

Region I Form 83
(Rev. January 5, 1982)

Dcts
8202020483

Enclosure 4

DCS No: 80244-820125
Date: January 25, 1982

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE--PNO-I-82- 09

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by IE staff on this date.

Facility: R.E. Ginna Nuclear Power Plant
DN 50-244
Ontario, New York

Licensee Emergency Classification:
_____ Notification of Unusual Event
_____ Alert
__X__ Site Area Emergency
_____ General Emergency
_____ Not Applicable

Subject: STEAM GENERATOR TUBE RUPTURE; SITE EMERGENCY

About 9:28 a.m., January 25, 1982, primary-to-secondary leakage was indicated in Steam Generator "B", and a reactor trip was initiated by low primary pressure. A Site Alert was declared and was upgraded to a Site Emergency at 10:50 a.m. An ENS report was made to the NRC about 9:35 a.m. The reactor containment was isolated. Safety injection and coolant charging were used to maintain primary inventory. No excessive core temperatures appear to have occurred. The pressurizer indicated full. Natural circulation cooling using Steam Generator "A" was attempted, but its effectiveness was questioned and a reactor coolant pump has been running in loop "A" since about 11:25 a.m.

At 12:50 p.m., the primary system was at 944 psig, core inlet temperature indicated 374 degrees, core thermocouple temperatures were about 371 degrees, and cooldown was proceeding at about 48 degrees/hour.

At 06 a.m., a 1.2 mR/hr reading was obtained about 1 mile south-southeast of the site; that location has returned to normal background. The highest onsite reading reported was 12 mR/hr at the information center; that location has also returned to normal background. There is no indication of a serious radiation hazard on or off the site.

The NRC Region I Administrator and a response team are enroute to the site. The resident inspector monitored the event on site and continues to do so. The licensee issued a press release. An NRC press release is planned. The Region I and Bethesda incident response centers are manned.

CONTACT: KISTER 488-1233 ALLAN 488-1340

DISTRIBUTION:

H. St.	MNBB	Phillips	E/W	Willste	Landow (Faxed by HQ)
Chairman Palladino	EDO	NRR	IE	NMSS	OIA
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Comm. Roberts	ELD			NSAC	ADM:Doc. Mgt. Br.
ACRS					
SECY	Regional Offices			TMI Resident Section	
CA				RI Resident Office	

Region I Form 83
(Rev. January 5, 1982)
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DCS No: 80244-820125
Date: January 25, 1982

PRELIMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE--PNO-I-82-09A

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by IE staff on this date.

Facility: R.E. Ginna Nuclear Power Plant
DN 50-244
Ontario, New York

Licensee Emergency Classification:
_____ Notification of Unusual Event
_____ Alert
__X__ Site Area Emergency
_____ General Emergency
_____ Not Applicable

Subject: STEAM GENERATOR TUBE RUPTURE; SITE EMERGENCY

About 2:15 p.m., the reactor coolant system (RCS) was at about 822 psig, 370 degrees. Boron sampling reported at 1:35 p.m. indicated 2250 ppm in the reactor coolant (RCS) and 1083 ppm in Steam Generator "B" (SG-B). A bubble was drawn in the pressurizer about 12:15 p.m., and pressurizer level was levelled out near 50 percent. Cooldown has been reduced to about 5 degrees per hour.

Early in the event, a Power Operated Relief Valve (PORV) failed open. The Pressurizer Relief Tank (PRT) rupture disc blew. The PORV block valve was shut. PRT Level was 92 percent at 2 p.m., and Containment Sump level was 9.3 ft. (about 8,000 gal.).

The NRC Region I Administrator and a response team are enroute to the site. The resident inspector monitored the event on site and continues to do so. The licensee issued a press release. An NRC press release is planned. The Region I and Bethesda incident response centers are manned.

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