PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

(215) 841-4000

August 20, 1979

Mr. Boyce H. Grier, Director
Office of Inspection and Enforcement
Region I
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Mr. Grier:

SUBJECT: Licensee Event Report Narrative Description

The following occurrence was reported to Mr. Greenman, Region I, Office of Inspection and Enforcement on June 26, 1979.

Reference:	Docket Number 50-277 50-278
Report No:	(Update) LER 2-79-32/1T-1
Report Date:	(Previous) July 11, 1979
Occurrence Date:	June 26, 1979
Facility:	Peach Bottom Atomic Power Station R.D. 1, Delta, PA 17314

Technical Specification Reference:

Technical Specification 6.9.2.a(9) requires reporting "performance of structures, systems, or components that require remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analysis in the safety analysis report . . ."

Description of the Event:

In response to the analysis required by IE Bulletin 79-02 Revision 1, 1243 seismic class 1 supports have been analyzed. As a result of this analysis, 92 supports have safety factors less than five with the design basis earthquake. Of these, eight have safety factors less than 1; i.e., may fail during the design basis earthquake. The eight supports are distributed as follows:

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1 on the Unit 2 HPCI System
1 on the Unit 3 HPCI System
1 on the Unit 2 RHR System
1 on the Unit 3 RHR System
2 on the Unit 2 Emergency Service Water System
1 on the Unit 3 Emergency Service Water System
1 on the Unit 3 Drywell Inerting System

These seismic supports are either single supports in a piping system or are widely separated as in the case of the Unit 2 emergency service water supports. The single HPCI support in each unit is for piping associated with the test flow path which is isolated from the injection flow path except during testing and is automatically isolated if initiation occurs during testing. One of two emergency service water system supports on Unit 2 and the one on Unit 3 are located on the service water side of a normally closed and locked manual valve connecting the emergency service water discharge and service water discharge. Failure in this location would not interfere with emergency service water delivery. The other Unit 2 emergency service water support is associated with one of the two redundant supply loops which could be isolated if necessary. The single RHR support on each unit is associated with a single subsystem and is not on piping common to the two subsystems. The single drywell inerting system support is on piping at a significant distance and several rigid supports outboard of the outer isolation valve which along with the inner isolation valves is normally closed. These deficiencies constitute less than one percent of the total supports analyzed.

Consequences of Event:

The calculational method used is conservative in that all stresses and loads are combined to obtain the total load during the design basis earthquake. Calculations indicate that these supports will withstand an operating basis earthquake.

In light of the conservative nature of the calculation, the Operation and Safety Review (O&SR) Committee has concluded that these deficiencies have a minimal safety significance and that the plant can continue operation.

Cause of Event:

The cause of this occurrence was inadequate initial support design.

Corrective Action:

Design of the necessary modifications has been completed. The Licensee Event Report dated July 11, 1979, committed to modifying these type supports by July 13, 1979. Completion of these modifications was delayed because design revisions were necessary due to installation interference problems and Q material availability. Modifications of all eight supports were completed on July 19, 1979.

Yours truly,

M. J. Cooney
Superintendent
Generation Division-Nuclear

Attachment

cc: Director, NRC - Office of Inspection and Enforcement
Mr. Norman M. Haller, NRC - Office of Management & Program Analysis