



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
611 RYAN PLAZA DRIVE, SUITE 1000  
ARLINGTON, TEXAS 76012

bcc to DAC:ADM:  
CENTRAL FILES  
PDR:HQ  
LPDR  
TIC  
NSIC

States

February 6, 1980

Docket No. 50-298

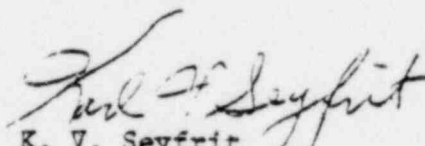
Nebraska Public Power District  
ATTN: J. M. Pilant, Director  
Licensing & Quality Assurance  
Post Office Box 499  
Columbus, Nebraska 68601

Gentlemen:

Enclosed is IE Bulletin No. 80-03, which requires action by you with regard to your reactor facility with an operating license or a construction permit.

Should you have questions regarding this Bulletin or the actions required of you, please contact this office.

Sincerely,

  
K. V. Seyfrit  
Director

Enclosures:

1. IE Bulletin No. 80-03
2. List of Recently Issued  
IE Bulletins

cc: L. C. Lessor, Superintendent  
Cooper Nuclear Station  
Post Office Box 98  
Brownville, Nebraska 68321

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
WASHINGTON, D.C. 20555

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DUPLICATE

IE Bulletin No. 80-03  
Date: February 6, 1980  
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LOSS OF CHARCOAL FROM STANDARD TYPE II, 2 INCH, TRAY ADSORBER CELLS

Description of Circumstances:

During preliminary leak tests of charcoal adsorber cells in certain ventilation systems at Sequoyah Nuclear Plant, it was determined that on certain adsorber cells the spacing between rivets securing the perforated screen to the casing was too great to ensure adequate contact between the casing and the screen, thus allowing charcoal to escape.

The problem was discovered when a visual inspection detected loose charcoal on the floor of the filter housings and on the outside horizontal surfaces of the adsorber cells. Loss of charcoal was also indicated by observation of light penetrating through the cells. Additional inspection revealed that the rivets securing the perforated screens to the cell casing were approximately six inches apart and the screen appeared to be sagging away from the casing between rivets.

The particular adsorber cells being tested at Sequoyah Nuclear Plant were Flanders Type II pre-1974 fabrication.

There is a possibility that design of adsorber cells with wide spacing between screen rivets may pass initial freon leak tests but degrade significantly during operation thus reducing the margin of safety during postulated accidents.

The responses from this Bulletin will be used by the NRC to evaluate need for more frequent inspection/testing.

For all power reactor facilities with an Operating License:

1. Determine if charcoal adsorber cells in use, or proposed for use, have the potential for a loss of charcoal incidental to handling, storage or use (as appropriate). Particular attention should be directed to examination of, a) rivet spacing resulting in separation of screen and cell housing, and b) adsorber cell or filter housing deformation causing loss of charcoal and/or channeling. Either of these items could result in a degraded filtration system incapable of performing its intended function. The preferred method of this determination is a visual inspection of the filter housing and adsorber cells as described in Section 5 of ANSI N510-1975. If this method is not feasible, state in the report required by Paragraph 4 how the determination was made.
2. For ESF filtration systems, any identified defective cells shall be replaced and the operability of the system (after cell replacement)

demonstrated by leak testing within 7 days. Preferred method of leak testing is as described in Regulatory Guide 1.52 and Section 12 of ANSI N510-1975.

3. For normal ventilation exhaust filtration systems which employ charcoal adsorber cells and for which radioactive removal efficiency has been assumed in determining compliance with the "as low as reasonably achievable" design criteria of 10 CFR 50, Appendix I, any identified defective cells shall be replaced as soon as possible but at least within 30 days. After replacement, the system should be demonstrated operable by leak testing within an additional 30 days. Preferred method of testing is as described in Regulatory Guide 1.140 and Section 12 of ANSI N510-1975.
4. Report in writing within 45 days of the date of this Bulletin the results of the determination required by Paragraph 1. The report shall include the type of cells employed (manufacturer and cell design), system containing the cells, observed cell condition (degradation/sagging) and a discussion of visual inspection procedure and results.

For all Power Reactor Facilities with a Construction Permit:

1. Visual inspection shall be conducted only if the charcoal adsorber cells have been purchased and shipment received. A representative number (approximately 5) of each type of cell design/manufacturer shall be visually inspected for such deficiencies as rivet spacing and screen/casing separation which could lead to loss of charcoal incidental to handling, storage, or use.
2. Report in writing within 45 days of the date of this Bulletin the results of the inspection required by Paragraph 1. The report shall include the type of cells (manufacturer and cell design), observed cell condition (degradation/sagging) and a discussion of the inspection procedure and results.

Reports shall be sent to the Director of the appropriate NRC Regional Office listed in Appendix D of 10 CFR 20 with a copy to the Director, Division of Fuel Facility and Materials Safety Inspection, Office of Inspection and Enforcement, USNRC, Washington, D.C. 20555.

Approved by GAO, B180225(R0072); clearance expires, 7/31/80. Approval was given under a blanket clearance specifically for identified generic problems.

RECENTLY ISSUED IE BULLETINS

Bulletin No.	Subject	Date Issued	Issued To
79-26	Boron Loss from BWR Control Blades	11/20/79	All BWR Power Reactor Facilities with an Operating License (OL) for action. All BWR's with a Construction Permit (CP) for information.
79-27	Loss of Non-Class-1-E Instrumentation and Control Power System Bus During Operation	11/30/79	All power reactor facilities holding Operating Licenses (OLs) and to those nearing licensing
79-28	Possible Malfunction of Namco Model EA 180 Limit Switches at Elevated Temperatures	12/7/79	All power reactor facilities with an Operating License (OL) or a Construction Permit (CP)
79-01B	Environmental Qualification of Class 1E Equipment	1/14/80	All power reactor facilities with an Operating License (OL)
80-01	Operability of ADS Valve Pneumatic Supply	1/11/80	All BWR power reactor facilities with an Operating License (OL)
80-02	Inadequate Quality Assurance for Nuclear	1/21/80	All BWR licenses with a Construction Permit (CP) or Operating License (OL)

Enclosure