

LICENSEE EVENT REPORT

CONTROL BLOCK

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

N J 0 C P I I 1 2 0 0 0 - 0 0 0 0 0 0 0 - 0 0 0 3 4 1 1 1 1 1 4 5  
 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 31 L 58  
 REPORT SOURCE L 6 0 5 0 0 0 1 2 1 9 7 0 1 1 3 1 8 0 8 0 2 2 9 8 0 9  
 DOCKET NUMBER 60 61 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

On January 31, 1980, during the reactor recirculation flow calibration  
 surveillance procedure, the as-found values of the recirc flow sensors  
 at zero percent simulated flow were out of tolerances on all APRM  
 channels except 5 and 6. Tests were performed to determine the signific-  
 ance of the out of tolerance value on respective APRM power/flow rod block  
 and scram trip points. The scram trip occurred at values higher than  
 Technical Specification requirements on 3 channels.

SYSTEM CODE I A 11 CAUSE CODE X 11 CAUSE SUBCODE X 13 COMPONENT CODE INSTRU 14 COMP SUBCODE E 15 VALVE SUBCODE Z 16  
 EVENT YEAR 8 0 21 22 SHUTDOWN METHOD Z 21 HOURS 0 0 0 0 37 40 ATTACHMENT SUBMITTED Y 23 NPRD-4 FORM SUB N 24 PRIME COMP. SUPPLIER N 25 COMPONENT MANUFACTURER  
 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

The cause of this occurrence is attributed to instrument draft. The safe-  
 ty significance of this event is considered minimal since the reactor is  
 not operated in the power/flow region of interest. The flow sensors were  
 recalibrated to all required flow rates.

FACILITY STATUS H 28 % POWER 0 0 0 29 OTHER STATUS NA 30 METHOD OF DISCOVERY B 31 DISCOVERY DESCRIPTION Surveillance Test 32  
 ACTIVITY CONTENT RELEASED OF RELEASE Z 33 Z 34 AMOUNT OF ACTIVITY NA 35 LOCATION OF RELEASE NA 36  
 PERSONNEL EXPOSURES NUMBER 0 0 0 37 TYPE Z 38 DESCRIPTION NA 39  
 PERSONNEL INJURIES NUMBER 0 0 0 40 DESCRIPTION NA 41  
 LOSS OF OR DAMAGE TO FACILITY TYPE Z 42 DESCRIPTION NA 43  
 PUBLICITY ISSUED DESCRIPTION Y 44 Weekly News Release 45

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Licensee Event Report  
Reportable Occurrence No. 50-219/80-6/3L

Report Date

February 29, 1980

Occurrence Date

January 31, 1980

Identification of Occurrence

Exceeding a limiting condition for operation as per Technical Specifications, table 3.1.1.A.8, when the recirc flow sensors safety settings were found to be less conservative than required.

This event is considered to be a reportable occurrence as defined in the Technical Specifications, paragraph 6.9.2.b.1.

Conditions Prior to Occurrence

The plant was shut down for a refueling/maintenance outage.

The reactor was subcritical.  
The reactor mode switch was locked in refuel.  
The cavity was flooded.

Description of Occurrence

On Thursday, January 31, 1980, during the Reactor Recirculation Flow Calibration (603.3.002) Surveillance procedure, the as-found values of the recirc flow sensors at zero percent simulated flow were out of tolerances on all APRM channels except 5 and 6. A Job Order (2721 1) was initiated to perform a surveillance test to determine the significance of out of tolerance value on respective APRM power/Flow rod block and scram trip points.

All APRM channels were tested using a simulated flow equivalent to the magnitude of zero error and increasing simulated power to the rod block and scram trip points.

1 - Rod block occurred at less than the Technical Specifications required value on all APRM channels for both as-found and as-left recirc flow data values.

- 2 - Scram trip occurred at  $\leq 34\%$  (Technical Specification required value) on all APRM channels except #2, #7, and #8 when using an as-found error value of recirc flow.
- 3 - Scram trip occurred at 36.5% for APRM #2, 35% for APRM #7, and 39% for APRM #8.

The out of tolerance readings were at zero percent simulated flow only. The plant will not experience power operation at this flow.

#### Apparent Cause of Occurrence

The cause of this occurrence is attributed to instrument drift.

#### Analysis of Occurrence

The Average Power Range Monitor (APRM) system consists of eight channels, four in each reactor protection system. Each channel averages the signals from eight Local Power Range Monitors (LPRM). The reactor is divided into four quadrants and each reactor protection system has an APRM in each quadrant.

The safety significance of this event is considered minimal since the reactor is not operated in this power/flow region.

#### Corrective Action

The flow sensors were recalibrated to all required flow rates.

#### Failure Data

Not Applicable.