

U.S. NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT

REGION III

Report No. 50-483/79-14

Docket No. 50-483

License No. CPPR-139

Licensee: Union Electric Company  
P.O. Box 149  
St. Louis, MO 63166

Facility Name: Callaway Nuclear Plant, Unit 1

Inspection At: Callaway Site, Callaway County, Missouri

Inspection Conducted: December 10-13, 1979

Inspector: *C. M. Erb*  
C. M. Erb

1/11/80

Approved by: *D. H. Danielson*  
D. H. Danielson, Chief  
Engineering Support Section 2

1/11/80

Inspection Summary

Inspection on December 10-13, 1979 (Report No. 50-483/79-14)

Areas Inspected: Observation of cleaning operations on internals and reactor cavity; observation welds in surge line; QA records for the pump and steam generator support columns; setting of reactor pressure vessel; previous inspection findings.

This inspection involved a total of 23 inspector hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

## DETAILS

### Persons Contacted (UECo)

- \*W. H. Weber, Manager, Nuclear Construction
- M. I. Doyne, General Superintendent
- \*R. L. Powers, Supervising Engineer, QA
- \*R. Veatch, Assistant Engineer, QA
- J. Gearhart, Engineer, QA

### Other Personnel

- H. J. Starr, Project Manager, Daniel International
- \*E. D. McFarland, Construction Engineering Manager, Daniel International
- \*J. A. Holland, Project QA Manager Daniel International
- \*W. L. Sykora, Assistant Project Manager, Daniel International
- J. Martinez, Site Manager, Westinghouse
- J. Long, Project Welding Engineer, Daniel International
- H. Kisner, Lead Mechanical QC Inspector, Daniel International

\*Denotes those present at the exit meeting.

### Licensee Actions on Previous Inspection Findings

(Closed) Unresolved item (483/79-03-01): An incorrect number appeared on a microfiche N-1 form supplied by Applied Engineering Company through Westinghouse. This applied to a Reheat Letdown Exchanger System and a corrected hard copy of the N-1 is now in the files. This unresolved matter is considered closed, but the microfiche must also be corrected in an expeditious manner.

### Functional or Program Areas Inspected

#### 1. Cleaning and Leveling of the Reactor Vessel

- a. Dry cleaning in the area of the transfer canal was being done. Fitting of the internals to the reactor vessel was being performed under clean room conditions in an acceptable manner.
- b. Installation check list, RPV/RBB01 for the reactor pressure vessel indicated steps where quality control verification had not been performed. One step in particular where 75% bearing under the vessel nozzle supports is required indicated only a check and signature by the field engineer. While Procedure SCP-210 states that engineering shall call out the hold points for QC, it appears that QC should insist on their signature and verification under the above conditions. The licensee agreed to review the need for QC inspection sign off on installation checklists. This is an unresolved item (483/79-14-01).

2. Installation and Welding of CRD Mechanism Lower Canopy Seal

Westinghouse (W) process specification PWR82127, Revision B, will be the basic requirement for installing the 78 CRD end welds. An automatic Astro-Arc welder will be qualified and a three channel strip recorder will record amperage, voltage, and speed. This weld will utilize a y-shaped insert and the base materials are stainless steel. Daniel Inc. is qualifying welders and their own procedure for this work. Applicable ASME codes for this work should be referenced in the Daniel procedures.

No items of noncompliance or deviations were identified.

3. Inspection of Surge Line Welds

Field Weld No. BB-01-F-001 in the 14" stainless steel surge line was examined. This is a class 1 weld and the weld has been ground to an acceptable contour for radiographic testing. Welds No. 1 and No. 2 were made to Procedure N-8-8-BA-2. The fit up on weld No. 4, which had not been made, was acceptable.

No items of noncompliance or deviations were identified.

4. QC Records Main Coolant Pump Supports and Steam Generator Supports

These supports were supplied by W and manufactured by Lamco. The QC records and certifications for the following parts were examined and found to be acceptable to the requirements of ASME Section III, 1974 Edition, Summer 1976 Addenda.

<u>No.</u>	<u>System</u>	<u>W Quality Release</u>
101064	Reactor Coolant Pump	33590
1126	Steam Generator Support	33599-1
1127	Steam Generator Support	33599-1

The above items were inspected at the source before shipment.

No items of noncompliance or deviations were identified.

Unresolved items

Unresolved items are matters about which more information is required in order to ascertain whether they are acceptable items, items of noncompliance, or deviations. An unresolved item disclosed during the inspection is discussed in Paragraph 1b.

#### Exit Interview

The inspector met with licensee and contractor representatives (denoted in Persons Contacted Paragraph) at the conclusion of the inspection on December 13, 1979. The inspector summarized the purpose, scope and findings identified during the inspection.