NRC FOR	M'366 U.S. NUCLEAR REGULATORY COMMISSION
X	CONTROL BLOCK:
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	REPORT L 6 0 5 0 0 0 3 6 6 7 0 1 2 9 8 0 8 0 2 1 3 8 0 9 SOURCE 60 61 DOCKET NUMBER 63 69 EVENT DATE 74 75 REPORT DATE 50 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
02	[On 1-29-80, while increasing reactor power by rod withdrawal, the operator noticed
03	the rod select matrix was not backlighted, and the RSCS was not initiating rod blocks
04	properly. The RSCS being inoperable is reportable per Tech Specs section 6.9.1.9.b.
0 5	[This event is not applicable to Unit 1. This is a non-repetitive occurrence. There
0 6	I were no effects upon public health and safety due to this event.
0 7	L
08	80
7 B	9 SYSTEM CAUSE CAUSE CAUSE COMPONENT CODE COMPONENT CODE SUBCODE SUBCODE SUBCODE COMPONENT CODE SUBCODE SUBCO
7 8	Image: Securital Report NO. OCCURRENCE REPORT REVISION NO. Image: Securital Report NO. Securital Report NO. Securital Report NO. Image: Securital Report NO. Securital Report NO. Securital NO.
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10	L. The RSCS did not function properly due to a design change which inadvertently removed
[1]1]	L the settle cycle input from the reactor manual control system to the group notch con-]
1 2	I trol mode of RSCS. A design change was performed to restore the settle cycle input.
13	No further corrective action is required.
14 7 8	9 9 80
15	STATUS 3 POWER OTHER STATUS 30 DISCOVERY DISCOVERY DESCRIPTION 32 C 30 10 0 6 29 NA A 44 45 46 Operator Observation 80
1 6 7 8	ACTIVITY CONTENT IELEASED OF RELEASE AMOUNT OF ACTIVITY 35 IZ 33 IZ 34 NA II 9 III 10 44 45 IIIIIIIIIIIIIIIIIIIIIIIIIIIIIIII
1 7 7 8	PERSONNEL EXPOSE DESCRIPTION (3) NUMBER TYPE DESCRIPTION (3) NA 11 12 PERSONNEL IN UNITES 13 0
	NUMBER DESCRIPTION (4) NA POOR ORIGINAL 80
19	LOSS OF OR DAMAGE TO FACILITY (43)
7 8	PUBLICITY ISSUED DESCRIPTION (5) 8002200472 NRC USE ONLY
1 8	9 10 NA 68 69 80.5
	NAME OF PREPARER_ C. L. Coggin, Supt. Plt. Eng. Serv. PHONE: 912-367-7781

LER #: 50-366/1980-008 Licensee: Georgia Power Company Facility: Edwin I. Hatch Docket #: 50-366

Narrative Report for LER 50-366/1980-008

On 1-29-80, with the reactor operating at 150 MWt in the run mode and while increasing reactor power by control rod withdrawal in rod group 7, the operator noticed that the rod select matrix was not backlighted, and the rod sequence control system was not initiating rod blocks properly. The RSCS was determined to be inoperable, and rod movement was stopped per Tech Specs section 3.1.4.2.b. RSCS being inoperable is a condition leading to operation in a degraded mode permitted by a limiting condition for operation, reportable per Tech Specs section 6.9.1.0.b. This event is not applicable to Unit 1. This is a non-repetitive occurrence. There were no effects upon public health and safety due to this event.

The RSCS did not function properly due to a design change which inadvertently removed the control rod settle cycle input from the reactor manual control system to the group notch control mode of RSCS. This design change was based on an FDDR which did not specify the addition of two wires internal to the new reactor manual control solid state timer. The omission of the internal wires caused the settle cycle input to be removed. A design change was performed to restore the settle cycle input to the original wiring configuration, allowing the RSCS to perform its intended function. No further corrective acton is required. The original design change will be re-installed, with the addition of the internal wires to the solid state timer, during an upcoming outage. The unit is now in full compliance with the requirements for RSCS operability, and no further reporting is required.



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