



UNITED STATES
NUCLEAR REGULATORY COMMISSION
ADVISORY COMMITTEE ON REACTOR SAFEGUARDS
WASHINGTON, D. C. 20555

SL-0239
CDR 8/28/79

July 5, 1979

Honorable Joseph M. Hendrie
Chairman
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: TWO-HUNDRED-THIRTIETH MEETING OF THE ADVISORY COMMITTEE ON
REACTOR SAFEGUARDS, JUNE 14-16, 1979 in WASHINGTON, DC

Dear Dr. Hendrie:

The Advisory Committee on Reactor Safeguards held its 230th meeting in Washington, D.C. on June 14-16, 1979. Items considered at this meeting are summarized below:

Three Mile Island Nuclear Station Unit 2 Accident and its Implications

The Committee continued its review of the accident at Three Mile Island Nuclear Station Unit 2 (TMI-2) and the implications of this accident for other nuclear power plants.

The Committee met with representatives of the NRC Staff, the Metropolitan Edison Company, and the Babcock and Wilcox Company to discuss the current status of the TMI-2 plant; evaluation of responses to NRC/IE Bulletins, NRC Orders, and ACRS recommendations emanating from consideration of the TMI-2 accident; and consideration of proposed improvements to equipment and procedures.

The Committee set up a new Subcommittee to consider the responses of vendors/utilities to IE Bulletins and NRC Orders resulting from the TMI-2 accident.

Further evaluation of the accident and its implications are planned.

Projects

The Committee deferred further action on its review of the application for an Operating License for the Sequoyah Nuclear Power Plant Units 1 and 2 until the NRC Staff informs the ACRS of its plans regarding changes to the plant design and/or operations that have been developed following the investigation of the TMI-2 accident.

7910170469

2223 343

The Committee also deferred further action on its review of the application for a Construction Permit for the Palo Verde Nuclear Generating Station Units 4 and 5 until the NRC Staff informs the ACRS of its plans regarding changes to the plant design that have been developed following the investigation of the TMI-2 accident.

The Committee has informed the Executive Director for Operations that the ACRS does not object to the NRC Staff's licensing the Millstone Nuclear Power Station Unit 2 to be operated at a power level of 2700 MWt.

The Committee noted the matter regarding alleged erroneous information provided to the Committee during its Operating License review of the Zimmer Nuclear Station and has requested that the NRC Staff report to the Committee regarding the resolution of this matter.

Miscellaneous

The Committee prepared a reply to Commissioner Gilinsky's inquiry regarding a comparison of the merits of stainless steel vs. zircaloy for nuclear fuel cladding. A copy has been sent to you and the other Commissioners.

The Committee requested that the NRC Staff inform the Committee regarding additional requirements resulting from evaluation of the TMI-2 accident that the NRC will impose on those facilities for which the Committee has written favorable Operating License reports, but for which Operating Licenses have not been issued.

The Committee concurred in the proposed revision (Revision 2) of Regulatory Guide 1.9, Selection, Design and Qualification of Diesel-Generator Units Used as Standby (Onsite) Electric Power Systems at Nuclear Power Plants.

Since the last report of ACRS activities, the following subcommittee meetings and site visits have been held:

Safeguards and Security, in Washington, D.C., on May 23, 1979

Evaluation of Licensee Event Reports, in Washington, D.C., on
May 24-25, 1979

TMI-2 Implications, in Washington, D.C. on May 31-June 1, 1979

Reliability and Probabilistic Assessment, in Washington, D.C., on
June 2, 1979

TMI-2, in Middletown, PA, on June 6-7, 1979

Reactor Fuel, in Washington, D.C., on June 11-12, 1979

Operating Reactors, in Washington, D.C., on June 13, 1979

Regulatory Activities, in Washington, D.C., on June 13, 1979

Two members of the Committee met on May 21-25, 1979, in Japan, with representatives of Japanese regulatory agencies, including the Committee on the Examination of Reactor Safety to discuss reactor safety policy and practice, and the cooperative emergency core cooling systems (ECCS) program.

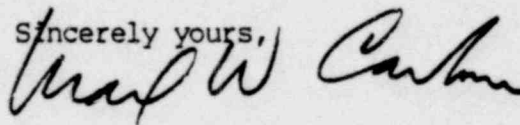
In addition to the above, representatives of the Committee visited the Babcock and Wilcox simulator at Lynchburg, VA on June 8, 1979, to observe simulator training.

The 231st meeting of the ACRS will be held in Washington, D.C., on July 12-14, 1979. The following matters are tentatively scheduled for discussion:

Three Mile Island Nuclear Station Unit 2 - status and implications

Bailly Generating Station - design of pilings

Sincerely yours,



Max W. Carbon
Chairman