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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION II  
101 MARIETTA ST., N.W., SUITE 3100  
ATLANTA, GEORGIA 30303

AUG 13 1979


In Reply Refer To:  
RII:JPO  
50-321, 50-366  
50-424, 50-425

Georgia Power Company  
Attn: J. H. Miller, Jr.  
Executive Vice President  
270 Peachtree Street, N. W.  
Atlanta, Georgia 30303

Gentlemen:

The enclosed Bulletin 79-21 is forwarded to you for information. No written response is required. If you desire additional information regarding this matter, please contact this office.

Sincerely,

  
For James P. O'Reilly  
Director

- Enclosures:
1. IE Bulletin No. 79-21 w/encl.
  2. List of IE Bulletins Issued  
in the Last 6 Months

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Georgia Power Company

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
OFFICE OF INSPECTION AND ENFORCEMENT  
WASHINGTON, D.C. 20555

August 13, 1979

IE Bulletin No. 79-21

TEMPERATURE EFFECTS ON LEVEL MEASUREMENTS

Description of Circumstances:

On June 22, 1979, Westinghouse Electric Corporation reported, to NRC, a potential substantial safety hazard under 10 CFR 21.

The report, Enclosure No. 1, addresses the effect of increased containment temperature on the reference leg water column and the resultant effect on the indicated steam generator water level. This effect would cause the indicated steam generator level to be higher than the actual level and could delay or prevent protection signals and could, also, provide erroneous information during post-accident monitoring. Enclosure No. 1 addresses only a Westinghouse steam generator reference leg water column; however, safety related liquid level measuring systems utilized on other steam generators and reactor coolant systems could be affected in a similar manner.

Actions To Be Taken By Licensees:

For all pressurized water power reactor facilities with an operating license:\*

1. Review the liquid level measuring systems within containment to determine if the signals are used to initiate safety actions or are used to provide post-accident monitoring information. Provide a description of systems that are so employed; a description of the type of reference leg shall be included, i.e., open column or sealed reference leg.
2. On those systems described in Item 1 above, evaluate the effect of post-accident ambient temperatures on the indicated water level to determine any change in indicated level relative to actual water level. This evaluation must include other sources of error including the effects of varying fluid pressure and flashing of reference leg to steam on the water level measurement. The results of this evaluation should be presented in a tabular form similar to Tables 1 and 2 of Enclosure 1.
3. Review all safety and control analyses throughout the reactor that the setpoints will in instrumentation, including these setpoints.

\*Boiling water reactors have been  
NRC to provide similar information

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