

## UNITED STATES NUCLEAR REGULATORY COMMISSION REGION II

101 MARIETTA STREET, N.W. ATLANTA, GEORGIA 30303

Report Nos.: 50-327/79-08 and 50-328/79-04

Licensee: Tennessee Valley Authority

500A Chestnut Street Tower II Chattanooga, Tennessee 37401

Facility Name: Sequoyah Nuclear Plant, Units 1 and 2

Docket Nos.: 50-327 and 50-328

License No.: CPPR-72 and CPPR-73

Inspection at Daisy, Tennessee

Inspector: 9 1 Um Don

Approved by I Conlon, Section Chief, RCESB

SUMMARY

Inspection on January 30 through February 1, 1979

### Areas Inspected

This routine, unannounced inspection involved 20 inspector-hours on-site in the areas of reactor coolant pressure boundary piping records and welding activities (Unit 2); safety-related piping handling and welding activities (Unit 2); preservice inspection records (Unit 1); safety-related structures records (Units 1 and 2).

#### Results

Of the four areas inspected, no apparent items of noncompliance or deviations were identified in three areas; one apparent item of noncompliance was found in one area (infraction-failure to follow procedure for handling and protection of stainless steel piping, paragraph 7.b). 2236 149

#### DETAILS

#### 1. Persons Contacted

#### Licensee Employees

- G. G. Stack, Project Manager, Construction
- \*R. W. Olson, Assistant Construction Engineer
- \*J. L. Smith, Assistant Construction Engineer
- \*J. M. Munns, QA Supervisor
- \*B. W. Farrell, Quality Control Records Unit Supervisor
- \*L. W. Jones, Welding Inspection Unit Supervisor
- \*R. L. Hamilton, QA Engineer
- \*E. C. Pendergrass, QC Records Unit
- W. E. Andrews, QA Staff Supervisor
- R. H. Daniels, Level III Examiner

Other licensee employees contacted included four construction craftsmen, three technicians and three office personnel.

\*Attended exit interview

#### Exit Interview

The inspection scope and findings were summarized on February 1, 1979, with those persons indicated in Paragraph 1 above. The noncompliance described in paragraph 7.b was discussed in detail.

## Licensee Action on Previous Inspection Findings

(Closed) Unresolved Item 50-327/78-41-02: Control of Linearity Calibrations for UT of Piping. This item concerned the fact that inspection personnel were not always performing daily linearity calibrations for UT of piping. The inspector verified that these daily calibrations are not required by the applicable Code (ASME Section V, 74S75). TVA, therefore, intends to leave the applicable procedure (UT-1) "as is". It contains only a 90-day calibration requirement in accordance with the code. TVA informed the inspector that, since minimal work was involved and added assurance was obtained, daily linearity calibrations would be performed. A further advantage is to eliminate confusion since daily calibrations are required by code for UT of pressure vessels. These actions are satisfactory and this item is closed.

#### 4. Unresolved Items

Unresolved items were not identified during this inspection.

## 5. Independent Inspection Effort

- a. (Units 1 and 2) The inspector conducted a general inspection of the containment and auxiliary buildings to observe construction progress and construction activities such as welding, material handling and control, housekeeping and storage.
- b. (Units 1 and 2) The inspector reviewed receiving records for Waste Gas Decay Tanks 1 through 9 which are common to both units. The inspector also observed four tanks in place. This inspection was performed in order to verify the name of the manufacturer since extensive weld defects were found in similar tanks manufactured in the same time frame for another site. The manufacturer was verified to be Delta Southern Company of Baton Rouge, Louisiana, which is not the manufacturer of the defective tanks.
- (Unit 1) The inspector reviewed pre-service UT inspection records for four Class A welds to determine if procedure and Code (ASME Section XI, 74S75) requirements were being met. Records were reviewed for 40 degree and L-wave inspection of loop welds RC-6, RC-4S1 and RC-4 and 45 degree and L-wave inspection of steam generator circumferential seam weld SGW-A2. Areas reviewed included examination results, equipment data, calibration data, identification of couplant, chart records, evaluation and disposition of findings, extent of inspection, calibration re-verification, accurate recording of indications, record reviews and control of temporary record storage. Within this same area, the inspector discussed inspector follow-up item 327/79-06-01 with TVA. This item concerned disposition of a significant UT indication found in weld RHRS-119. Power plant maintenance branch has recommended repair of the weld; however, the final decision to repair has not been made. This item remains open.
- d. (Unit 2) The inspector reviewed welding above the root layer on reactor coolant pressure boundary and other safety-related piping to determine if procedure and code requirements were being met. The applicable code is the ANSI standard B31.7, 1969 edition and the 1970 addenda. Areas reviewed included weld identification, use of applicable welding procedure, use of specified weld material, physical appearance of weld and documentation held at the work site. The following welds were observed:

Weld Number	Class	Size	Type of Weld	
2CX02642	В	1"x.250"	Socket	
2CX02643	В	1"x.250"	Socket	
2CX01198	A	2"x.343"	Socket	

Weld Number	Class	Size	Type of Weld
(Continued) 2CX01199	A	2"x.343"	Socket
2CX01043	A	2"x.343"	Socket
2CX01044	A	2"x.343"	Socket
2CX01094B	A	2"x.343"	Socket
2CX01316	В	2"x.343"	Butt
2CS00155	В	8"x.322"	Butt

No items of noncompliance or deviations were identified.

# 6. Reactor Coolant Pressure Boundary Piping-Review of Quality Records (Unit 2)

Procurement, installation and inspection of components (e.g., pipe spools, fittings, pump/valve casings, etc.) in the reactor coolant pressure boundary piping system is controlled by the code delineated in paragraph 5.d, and approved QA/QC site procedures/specifications.

Records of the following items were selected for review to ascertain whether they (records) were in conformince with established requirements and reflect material/component properties consistent with applicable requirements. Components selected for this undertaking were as follows:

Item		System
Loop 2-1		RC
Loop 2-5		RC
Loop 1-3		RC
Loop 1-4		RC
Loop 4-1		RC
Thermowells	#TEN/RCPCTW-09	RC
Thermowells	#TEN/RCPCTW-10	RC
Thermowells	#TEN/RCPCTW-11	RC
Thermowells	The same of the sa	RC
2SI-95		SI
2SI-101		SI
2SI-123		SI
2RC-16		RC
2RC-17		RC
2RC-20		RC

The review effort was directed in the following areas: material test reports/certifications; NSSS manufacturer's quality releases; licensee receipt inspection reports; and, nonconformance/deviations as applicable.

Records of piping storage inspections performed back to July 1978 were reviewed to ascertain whether inspection frequency and storage level requirements were being maintained.

QA surveillance/audit reports numbers SN-M-78-06 and SN-M-78-07 relative to piping were reviewed to ascertain whether applicable audit requirements were being met in the following areas: scope and frequency specified; identified deficiencies resolved and that the corrective action taken was sufficient to preclude identified problems from recurring.

## 7. Safety-Related Piping-Observation of Work and Work Activities (Unit 2)

- a. The inspector observed handling of eight inch containment spray piping on either side of weld 2CS00155 to determine if code and procedure requirements were being met. The applicable code is delineated in paragraph 5.d. The applicable site procedure is process specification number 4.M.1.1(b) of December 20, 1973.
- b. On January 31, 1979, the inspector observed stainless steel containment spray piping in the containment dome area with the following examples of noncompliance with the above process specification:
  - (1) Pipe end caps missing-three examples.
  - (2) Uncovered wire rope around pipes-two examples.
  - (3) Pipe placed against unpainted/uncovered carbon steel stands-four examples.
  - (4) Spray nozzle bosses uncovered-approximately one-third of bosses observed.

This is in noncompliance with 10 CFR 50, Appendix B, Criterion V as implemented by the FSAR, paragraph 17.1A.5. This is Infraction 328/79-04-01.

No items of noncompliance or deviations were identified, except as identified in paragraph 7.b.

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