NRC PDR

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

APRIL 20 1979

Docket No.: 50-334

LICENSEE: Duquesne Light Company

FACILITY: Beaver Valley Power Station, Unit No. 1

SUBJECT: MEETING SUMMARY - PIPE STRESS ANALYSIS

The licensee and Stone and Webster Engineering Corporation met with the staff on April 11 and 12, 1979 in Boston to discuss the status of the reanalysis efforts and to review the completed packages to-date. The list of attendees is attached as Enclosure 1.

The review centered on four completed packages (pipe stresses, hanger loads, base plate and bolts) and on seven partially completed packages. These packages are listed in Enclosure 2. The review of the systems being reanalyzed resulted in a general but incomplete agreement on the total systems requiring reanalysis. Stone & Webster agreed to provide a complete list of Category 1 piping, a full set of P&ID's, and a line list of all those analyzed by Shock 2, and those safety related. While it was originally thought that about 150 lines were analyzed by Shock 2, S&W has discovered an additional 150 runs which may include earlier duplicates. Therefore, the total runs using Shock 2 is somewhere between 150 and 300. S&W will survey these additional computer analyses to finalize a composite list.

The review of the pipe stress and hanger evaluation packages revealed several areas which were brought to the attention of the licensee. The packages need a better quality control; the licensee noted that they had accepted only one of the completed packages and quality control was a function that the licensee would perform. The review seems to be taking a great deal of time. The licensee has noted this to S&W in an effort to maintain their schedule of review. The largest number of problem areas identified from this limited review appears to be on the loading of the anchor bolts of pipe supports. With the information and data being used in the analyses, the seismic effects results in increased loadings on the supports and bolts.

Five problem sets were identified to the licensee as a request for additional information. These problem sets will be used by EG&G-Idaho

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to perform audit calculations to support our safety evaluation reports. The information requested is listed in Enclosure 3. The problem sets requested are as follows:

Run	Identification
783	Steam Generator Feedwater Line on 1A Steam Generator
1200	Pressurizer Spray Line
203	Auxilliary Feedwater Suction Line
204	Auxilliary Feedwater Pump Discharge
255A	Residual Heat Removal Lines

In the conclusion of the meeting, the licensee was requested to advise the NRC at the earliest possible time if a modified amplified response spectra was to be used in the Beaver Valley review. The licensee was also requested to supply a weekly status of runs report.

Dave Wigginton

Dave Wigginton
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures: As stated

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Enclosure 1

ATTENDEE LIST

BEAVER VALLEY - 1 IN BOSTON

MEETING WITH NRC, APRIL 11, 1979

Name	Organization
J. M. Cumiskey	S&W
Don King	S&W
W. J. L. Kennedy	S&W
N. R. Tonet	DLC
J. J. Carey	DLC
D. K. Morton	EG&G Idaho
W. T. Russell	NRC
J. M. Giannelli	NRC
R. G. LaGrange	NRC
D. L. Wigginton	NRC
E. V. Imero	NRC

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Enclosure 2

April 11 & 12, 1979 REVIEW - BEAVER VALLEY UNIT #1

Complete Packages

1) Auxilliary Feedwater Pump Discharge Lines
2) Recirculation Heat Exchanger - River Water Outlet
3) Low Head Safety Injection Pump Suction Lines from Sump (Including Sump Cross Tie)
4) Reactor Coolant Piping and 8" Crossover Line Between Main Loop Isolation Valves

Partial Packages

1) Pressurizer Spray
2) Steam Generator Feedwater - 1A Generator
3) Steam Generator Feedwater - 1B Generator
4) Steam Generator Feedwater - 1C Generator
5) Reactor Coolant Piping - 1A Loop
6) Residual Heat Removal
7) Auxilliary Feedwater Lines Suction

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Enclosure 3

The following engineering data is required to perform an ANSI B31.1 audit calculation:

1. List of design and operating temperatures and pressure.
2. Detailed piping drawings and/or isometrics indicating the geometry of the piping system along with all necessary dimensions (including bend radii) and weld locations.
3. List of nominal pipe sizes, schedules, weights, insulation weights, and materials.
4. List of pipe support types, locations, stiffnesses, and preloads (spring hanger hot loads).
5. List of valve locations, weights, lengths, and eccentricities (center of gravity locations)
6. List of any anchor movements (thermal, seismic, etc.) and locations.
7. OBE and SSE response spectra.
8. Description of any special fittings or components present on the piping system.
9. Stress summary based on Shock II/Nupipe results.

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