ARKANSAS POWER & LIGHT COMPANY
POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000

May 21, 1979

1-059-29

Director of Nuclear Reactor Regulation
ATTN: Mr. Robert W. Reid, Chief
Operating Reactors Branch #4
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Subject: Arkansas Nuclear One-Unit 1
Docket No. 50-313
License No. DPR-51
May 11, 1979 Letter Update
(File: 1510)

Gentlemen:

To provide a continued update to the NRC staff regarding progress made in implementing the commitments made in our May 11, 1979 letter and to address comments of the staff as a result of the May 17, 1979 Order, the following is provided.

Item (a)

The Emergency Feedwater (EFW) system capacity has been verified to be consistent with Technical Specification requirements and in accordance with recent B&W accident analyses. Specifically, previous requirements for EFW total capacity were quoted as 624 GPM, the latest B&W analyses requires 550 GPM while actual test data taken during initial plant startup indicate a minimum 672 GPM capability during accident condition assumptions. Therefore, the EFW system flow capability is consistent with accident analysis assumptions. All of the above information except the B&W analysis flow assumptions, was provided in detail in our May 17, 1979 letter.

As a result of staff concerns, we will perform a test on the EFW system to demonstrate Integrated Control System (ICS) override capability and controllability of EFW flow independent of ICS, verify the minimum EFW flow capability is conservative with respect to the accident analyses, and verify the operability of the EFW flow indication devices described in our May 17, 1979 letter. This test procedure is currently being developed and will be telecopied to you on May 22, 1979. The procedure will be available for inspection by your I&E inspector.

2047 199

To provide documentation on the operator training program, the following information is submitted.

The operator training program has included:

(a) A 1½ hour lecture given by the plant training section on the chronology of the TMI-2 incident and interim recommendations on proper operator actions such as those given by B&W.

(b) B&W simulator training which involved approximately 1½ to 2 hours of classroom training on the sequence of events at TMI-2, a 1/2 hour demonstration of the significant portions of the event, and 2-2½ hours of hands-on training in a TMI scenario. Seventeen individuals have completed this program.

The remaining ten people will complete the training on May 22 and 23, 1979.

(c) Approximately 1-1½ hours of training on emergency procedures that have changed as a result of the TMI incident. Emphasis was placed on the immediate actions and the changed portions of the procedures. This lecture was given by the plant training staff.

(d) Approximately 4 hours of training on the guidelines received from B&W following their small breaks analysis, the procedural changes made as a result of the small breaks analysis, and the design changes that are being made to the plant. This training was given by the plant training staff.

The following evaluation methods have been used to measure the effectiveness of the training program.

(a) A sixteen question exam was given to the operators. Seventeen individuals received scores above 90%. None of the operators received a score of less than 80%. The exam was reviewed by a member of the NRC Audit Team onsite on May 21, 1979.

(b) NUS Inc. conducted an audit of the training program and its effectiveness. This included interviews with a cross section of six operators. The initial results of the audit were favorable. The results will be made available to the NRC Audit Team.

(c) B&W also conducted an audit of the training program and interviewed six operators. The initial results of this audit were favorable. The results will be made available to the NRC Audit Team.

Very truly yours,

David C. Trimble

David C. Trimble
Manager, Licensing

DCT/ew

2047 201