

LICENSEE EVENT REPORT

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK: | | | | | | | | | | ①

0 1 | 0 | H | D | B | S | 1 | 2 | 0 | 0 | - | 0 | 0 | N | P | F | - | 0 | 3 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | | | 5
 7 8 9 14 15 25 26 30 57 58
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT 58

CON'T
 0 1 | R | P | T | S | O | U | R | C | E | L | 6 | 0 | 5 | 0 | - | 0 | 3 | 4 | 6 | 7 | 0 | 4 | 2 | 9 | 7 | 8 | 8 | 0 | 7 | 2 | 8 | 7 | 8 | 9
 7 8 60 61 68 69 74 75 80
 REPORT SOURCE DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES ⑩

0 2 | During a transient on 4/29/78 while the unit was in the process of shutting down for a
 0 3 | maintenance outage and was operating at 20% reactor power, Reactor Coolant System
 0 4 | (RCS) pressure dropped to approximately 1620 psig and Safety Features Actuation System
 0 5 | (SFAS) Channels 1 and 3 tripped at 0553 hours on RCS low pressure. This initiated
 0 6 | incident levels 1 and 2. There was no danger to the health and safety of the public
 0 7 | or unit personnel. No actual loss of coolant accident occurred. Equipment actuated
 0 8 | as designed to inject water into the RCS. (NP-30-78-01) 80

1 | 0 9 | S | F | 11 | A | 12 | Z | Z | Z | Z | Z | Z | 14 | Z | 15 | Z | 16 | 17 | 7 | 8 | 18 | 19 | A | 20 | C | 21 | 0 | 6 | 6 | 22 | 9 | 9 | X | 30 | 1 | 32 | X | 33 | X | 34 | 35 | 36 | 0 | 2 | 4 | 37 | 40 | Y | 23 | N | 24 | Z | 25 | Z | Z | Z | Z | 26 | 44 | 47
 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE
 LER-RO REPORT NUMBER EVENT YEAR SEQUENTIAL REPORT NO. OCCURRENCE CODE REPORT TYPE REVISION NO.
 ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS ATTACHMENT SUBMITTED NPRD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS ⑳

1 10 | The cause of this high pressure injection actuation was a low Reactor Coolant System
 1 11 | pressure trip resulting from the opening of feedwater valves. The cause of the feed-
 1 12 | water valves opening was due both to the sensitivity of the feedwater system while
 1 13 | repetitively transferring from flow control to level control of steam generator level
 1 14 | and improper operator response to the transient. 80

1 5 | D | 28 | 0 | 2 | 0 | 29 | NA | A | 31 | NA | DISCOVERY DESCRIPTION 32
 FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY
 1 6 | Z | 3 | 12 | 34 | NA | NA | LOCATION OF RELEASE 36
 ACTIVITY RELEASED OF RELEASE AMOUNT OF ACTIVITY
 1 7 | 0 | 0 | 0 | 37 | Z | 38 | NA | PERSONNEL INJURIES DESCRIPTION 39
 PERSON NUMBER SURES TYPE DESCRIPTION
 1 8 | 0 | 0 | 0 | 40 | NA | LOSS OF OR DAMAGE TO FACILITY DESCRIPTION 41
 PERSONNEL INJURIES NUMBER DESCRIPTION
 1 9 | Z | 42 | NA | LOSS OF OR DAMAGE TO FACILITY DESCRIPTION 43
 TYPE DESCRIPTION
 2 0 | N | 44 | NA | PUBLICITY ISSUED DESCRIPTION 45
 PERSON NUMBER DESCRIPTION

7903050 342

NRC USE ONLY

DVR 78-071

NAME OF PREPARER Stan Batch

PHONE: 419-259-5000, Ext. 251