

SOUTH CAROLINA ELECTRIC & GAS COMPANY

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VICE PRESIDENT AND GROUP EXECUTIVE
SPECIAL SERVICES AND PURCHASING

March 12, 1979

United States Nuclear Regulatory Commission
Attn: Mr. James P. O'Reilly
Director
Region II
101 Marietta Street, NW
Atlanta, Georgia 30303

Subject: V. C. Summer Nuclear Station
Unit #1
Reportable Items In Accordance
With 10CFR50.55(e)

Gentlemen:

On February 8, 1979, SCE&G identified to Region II concerns with the quality of accepted ASME code pipe radiographs. This letter is to provide an interim report on this matter and to update the status of previously identified items.

During routine SCE&G/QA surveillance of radiographic film packets, a number of packets were found which the SCE&G Level III NDE examiner evaluated as unacceptable because of documentation errors, technique or sensitivity problems and/or potential weld quality problems. The number of packets considered unacceptable by SCE&G is in excess of that which could be explained by difference of interpretation.

Our evaluation to date indicates that the condition was the result of the level of pipe welding at this point on the Project together with a higher than desired number of repair radiographs resulting in pressure on interpreters to not fall behind. SCE&G also considers selection of a film type that provided marginal sensitivity a strong contributor even though the film selected always met Code requirements. This position is substantiated by the large number of joints that have subsequently been resolved as acceptable after being reshot with a more sensitive film. Many of the differences of opinion among Level III interpreters, are also being resolved with the reshots on the more sensitive film.

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As immediate corrective action, a review of all film packets accepted to date was undertaken. Approximately 500 packets out of 1100 have been reviewed. Of these, slightly over 25% have been judged questionable for technique, sensitivity or potential weld quality. Each of these questionable packets is presently being resolved by a team composed of the SCE&G Level III and the Level III of the Code Stamp Holder. Resolution includes a large number of reshots, physical inspection of the joint, etc. To date, no welds have been reworked.

As part of the long term corrective action, until a change is justified based on results, SCE&G/QA will review 100% of all accepted safety related radiographic film packets. Any reshots will be taken using a film specified by SCE&G. More attention will be devoted to initial film selection. Efforts have been on-going for some time to reduce the number of repair radiographs.

We are confident that our initiated action will assure relevant defect-free weld joints documented by acceptable radiographic film. A final report will be issued after the re-evaluation is complete.

Our update of previously reported items is as follows:

1. Emergency Diesel Generator Deficiencies

Under a letter dated March 6, 1979, an SCE&G/QA release to work was transmitted to the Diesel Generator supplier for fluorescent P.T. inspection of rocker arms (and any necessary repair) and replacement of lube oil headers. The supplier is now on-site and has to date, removed and tested all rocker arms. No rocker arms required replacement but several were ground to remove rough areas. Lube oil headers have been removed and replacement is pending arrival of hardware.

2. Rockwell Valve Operator Qualification

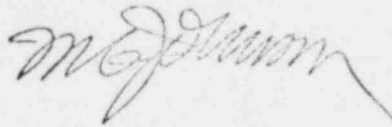
On February 14, 1979, representatives of SCE&G/QA participated in the final inspection of the operators that had been returned for rework. Surveillance was performed to confirm that all agreed material upgrading measures had been accomplished and certification was available to substantiate the success of the program. The returned operators were released for shipment contingent on the supplier's field service representative installing limit switches when received. In this regard, SCE&G has worked with Rockwell and the switch manufacturer directly to assure necessary QA program controls sufficient that the switches installed will be fully qualified.

As an item of information, in our original June 16, 1978 report on this problem of subvender qualification, we apprised you of a concern with incorrect spring material in Anchor Darling valves. Anchor Darling has confirmed by testing in a report dated February 14, 1979, that the replaced springs met design requirement and therefore, as indicated to you, the item did not constitute a reportable condition.

Since all SCE&G action on this item is complete with the exception of on-Site installation of the limit switches, we consider this the final report on this item.

Documentation of all of the above items is available for your review during subsequent inspection. Please feel free to contact us if we can provide additional information.

Very truly yours,



DAN/MCJ/jls

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