NRC FORM 366 (7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: [| | | | |] (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

Line	Field	Entry
0 1	LICENSEE CODE (2)	O H D B S 1
	LICENSE NUMBER (3)	Ø Ø - Ø Ø N P F - Ø 3
	LICENSE TYPE (4)	4 1 1 1 1
	CAT (5)	
CON'T 0 1	REPORT SOURCE (6)	L
	DOCKET NUMBER (7)	Ø 5 Ø - Ø 3 4 6
	EVENT DATE (8)	Ø 2 Ø 5 7 9
	REPORT DATE (9)	Ø 2 2 8 7 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 It was noted on February 5, 1979 by the Nuclear Regulatory Commission that a required
0 3 thirty day written report had not been submitted. As a result of surveillance testing
0 4 conducted on July 21, 1978, ten of the eighteen main steam safety valves were found
0 5 outside the setpoint range. This placed the unit in the Action Statement of Tech
0 6 Spec 3.7.1.1. The valves were returned to operable status within four hours. There
0 7 was no danger to the health and safety of the public or unit personnel. The valves
0 8 would still have served their intended function. (NP-33-79-25)

Line	Field	Entry
0 9	SYSTEM CODE (11)	C C
	CAUSE CODE (12)	E
	CAUSE SUBCODE (13)	B
	COMPONENT CODE (14)	V A L V E X
	COMP. SUBCODE (15)	P
	VALVE SUBCODE (16)	B
	LER/RO REPORT NUMBER (17): EVENT YEAR	7 9
	SEQUENTIAL REPORT NO.	Ø 2 Ø
	OCCURRENCE CODE	Ø 3
	REPORT TYPE	L
	REVISION NO.	Ø
	ACTION TAKEN (18)	E
	FUTURE ACTION (19)	Z
	EFFECT ON PLANT (20)	Z
	SHUTDOWN METHOD (21)	Z
	HOURS (22)	Ø Ø Ø Ø
	ATTACHMENT SUBMITTED (23)	Y
	NPRD-4 FORM SUB. (24)	Y
	PRIME COMP. SUPPLIER (25)	N
	COMPONENT MANUFACTURER (26)	D 2 4 3

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 The cause of the variation is believed to be due to changing ambient conditions at the
1 1 location of the valves. Subsequent lifting of safety valves from numerous unit trips
1 2 may have also affected their lift setpoint. The valves were adjusted and retested
1 3 satisfactorily and declared operable by 1600 hours on July 21, 1978. Facility Change
1 4 Requests have been issued which will help control ambient conditions.

Line	Field	Entry
1 5	FACILITY STATUS (28)	C
	% POWER (29)	Ø Ø Ø
	OTHER STATUS (30)	NA
	METHOD OF DISCOVERY (31)	B
	DISCOVERY DESCRIPTION (32)	Surveillance Test
1 6	ACTIVITY RELEASED (33)	Z
	CONTENT OF RELEASE (34)	Z
	AMOUNT OF ACTIVITY (35)	NA
	LOCATION OF RELEASE (36)	NA
1 7	PERSONNEL EXPOSURES NUMBER (37)	Ø Ø Ø
	TYPE (38)	Z
	DESCRIPTION (39)	NA
1 8	PERSONNEL INJURIES NUMBER (40)	Ø Ø Ø
	DESCRIPTION (41)	NA
1 9	LOSS OF OR DAMAGE TO FACILITY TYPE (42)	Z
	DESCRIPTION (43)	NA
2 0	PUBLICITY ISSUED (44)	N
	DESCRIPTION (45)	NA

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NRC USE ONLY

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TOLEDO EDISON COMPANY
DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE
SUPPLEMETAL INFORMATION FOR LER NP-33-79-25

DATE OF EVENT: February 5, 1979

FACILITY: Davis-Besse Unit 1

IDENTIFICATION OF OCCURRENCE: Main Steam Safety Valves were tested and found to be outside of ± 1% design setpoint range

Conditions Prior to Occurrence: On July 21, 1978, the unit was in Mode 3, with Power (MWT) = 0, and load (Gross MWE) = 0

Description of Occurrence: It was noted on February 5, 1979, by the Nuclear Regulatory Commission that a required thirty day written report covering the out of tolerance setpoint of ten main steam safety valves had not been submitted.

As a result of surveillance testing conducted on July 21, 1978, ten of the eighteen valves during the initial lift were found to be outside of ± 1% design setpoint range.

This placed the unit in the Action Statement of Technical Specification 3.7.1.1, which required the valves to be operable in Modes 1, 2, and 3. The Action Statement required the valves to be restored to operable status within four hours or the high flux trip setpoint be reduced or the unit be in Hot Shutdown (Mode 4) within six hours and Cold Shtudown within thirty hours. The valves were returned to operable status within four hours.

Designation of Apparent Cause of Occurrence: The apparent cause of the slight variation in relief valve lift pressure is believed to be due to changing ambient conditions at the location of the valves. The ambient conditions at the location of the valves were different from the last test conducted in January, 1978.

Subsequent lifting of safety valves from numerous unit trips may have also affected their lift setpoint.

Analysis of Occurrence: There was no danger to the health and safety of the public or to unit personnel. The unit was being started up at the time of the conduct of the test. The valves would still have served their intended function of relieving pressure if a unit trip had occurred.

Corrective Action: The valves were adjusted and retested per Maintenance Work Order 78-800 and the last was declared operable by 1600 hours on July 21, 1978.

Facility Change Requests 77-322 and 77-319 have been issued for implementation to install hoods on the main steam safety valves exhaust stack for protection during the winter months and provide ventilation during the summer months. Both FCRs will help control ambient conditions. Temporary ventilation will be provided if ambient temperature becomes excessive.

Failure Data: There have been two previously reported occurrences of incorrect safety valve lift setpoints. See Licensee Event Reports NP-33-77-117 and NP-33-78-145.

LER #79-020