

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555



AUG 0 8 1979

Docket Nos. 50-373 and 50-374

Mr. Byron Lee, Jr.
Vice President
Commonwealth Edison Company
P. O. Box 767
Chicago, Illinois 60690

Dear Mr. Lee:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION CONCERNING LA SALLE COUNTY STATION, UNITS 1 & 2

As a result of our continuing review of the La Salle Final Safety Analysis Report, we find we need additional information in the area of core performance to continue our evaluation. The specific information required is listed in the Enclosure.

Please inform us after receipt of this letter of the date you can supply the requested information.

Please contact us if you desire any discussions or clarification of the information requested.

Sincerely,

Olan D. Parr, Chief Light Water Reactors Branch No. 3 Division of Project Management

Enclosure: As Stated

cc w/enclosure: See next page

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Mr. Byron Lee, Jr.

cc: Richard E. Powell, Esq. Isham, Lincoln & Beale One First National Plaza Chicago, Illinois 60670

> Dean Hansell, Esq. Assistant Attorney General State of Illinois 188 West Randolph Street Suite 2315 Chicago, Illinois 60601

Mr. Roger Walker, Resident Inspector U. S. Nuclear Regulatory Commission P. O. Box 737 Streator, Illinois 61364

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ENCLOSURE

REQUEST FOR ADDITIONAL INFORMATION

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230.0 CORE PERFORMANCE BRANCH

- The requirements for analyzing the asymmetric hydraulic horizontal load have been changed by the A-2 Task Force (Asymmetric Blowdown Loads on Reactor Primary Coolant System) to include BWRs. To satisfy the new requirements La Salle should perform a plant specific analysis of their plant using the best available methodology and criteria for your design. At this time, La Salle should provide the results of an analysis which shows that their fuel assemblies can withstand the asymmetric loadings. La Salle should also commit to perform a further evaluation of their fuel following our approval of the generic analysis method should the methodology or criteria developed from this task warrant such reevaluation.
- 231.27 General Electric has recently informed us that the prior control rod design limit of 40 percent burnup of the B-10 content will lead to failure for most of the control rods. A new design limit of 32 percent burnup of the B-10 content has been recommended and is under review by us. Confirm that La Salle is aware of this proposed change in control rod design life.