OPERATING DATA REPORT

DOCKET NO. 50-267
DATE 790601
COMPLETED BY J. W. Gahm
TELEPHONE (303) 785-2253

OPERATING STATUS

NOTES

1. Unit Name: Fort St. Vrain, Unit No. 1
2. Reporting Period: 790501 to 790531
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
   None
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: Nuclear Regulatory Commission restriction (70%) pending resolution of certain Final Safety Analysis Report and Technical Specification bases discrepancies. This unit is in the power ascension phase of startup testing.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	3,623	---------
12. Number of Hours Reactor Was Critical	28.3	817.7	14,291.4
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	665.3	8,507.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	23	313,092	3,516,090
17. Gross Electrical Energy Generated (MWH)	0	109,852	1,058,122
18. Net Electrical Energy Generated (MWH)	0	101,177	952,628
19. Unit Service Factor	N/A	N/A	N/A
20. Unit Availability Factor	N/A	N/A	N/A
21. Unit Capacity Factor (Using MDC Net)	N/A	N/A	N/A
22. Unit Capacity Factor (Using DER Net)	N/A	N/A	N/A
23. Unit Forced Outage Rate	N/A	N/A	N/A

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    Shutdown for Refueling on 2-1-79
25. If Shut Down at End of Report Period, Estimated Date of Startup: 6-15-79
26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	740201	740131
INITIAL ELECTRICITY	7612	761211
COMMERCIAL OPERATION	------	------

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UNIT SHUTDOWNS AND POWER REDUCTONS

DOCKET NO. 50-267
UNIT NAME Fort St. Vrain, Unit No. 1
DATE 790601
COMPLETED BY J. W. Gahm
TELEPHONE (303) 785-2253

REPORT MONTH May, 1979

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
79-04	790201	F	744	A	4	79-03/03-L-0	HH	PUMPXX	Excessive gland leakage on "B" boiler feed pump concurrent with inoperable "C" boiler feed pump necessitated controlled reactor shutdown on February 1, 1979, per LCO 4.3.2. Scheduled reactor shutdown for refueling was changed from March 1, 1979, to February 1, 1979. Thus shutdown number 79-04 has extended through February, March, April, and May, 1979.

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SUMMARY: Plant shutdown for scheduled refueling and turbine generator overhaul entire month.

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267

Unit Fort St. Vrain, Unit No. 1

Date 790601

Completed By J. W. Gahm

Telephone (303) 785-2253

Month May, 1979

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

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*Generator on line but no net generation.

REFUELING INFORMATION

1. Name of Facility.	Fort St. Vrain, Unit No. 1
2. Scheduled date for next refueling shutdown.	February 1, 1979
3. Scheduled date for restart following refueling.	June 1, 1979
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	Yes*
If answer is yes, what, in general, will these be?	To facilitate insertion of eight fuel test elements.
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10CFR Section 50.59)?	*NOTE: If Technical Specification change approval if not received from Nuclear Regulatory Commission in time for refueling, then the answer to #4 is NO, and the reload fuel and graphite design have been reviewed.
If no such review has taken place, when is it scheduled?	
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	January 9, 1978
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	Eight test fuel elements to allow: 1) Different fuel particle design. 2) To qualify near isotropic graphite.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements. b) 245 spent HTGR fuel elements.
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one third of core (approximately 500 HTGR elements). No change is planned.

REFUELING INFORMATION (CONTINUED)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1986 under the Three Party Agreement (Contract AT (04-3)-633) between DOE, Public Service Company of Colorado (PSCo), and General Atomic Company.*

*The 1986 date is based on the understanding that spent fuel discharged during the term of the Three Party Agreement will be shipped to the Idaho National Engineering Laboratory for storage by DOE at the Idaho Chemical Processing Plant (ICPP). The storage capacity has evidently been sized to accomodate fuel which is expected to be discharged during the eight year period covered by the Three Party Agreement.